

# OFFICE OF CIVILIAN RADIOACTIVE WASTE MANAGEMENT

## DESIGN CALCULATION OR ANALYSIS COVER SHEET

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## 1. PURPOSE

The purpose of this calculation is to determine the radiolytic production of nitric acid and other chemical species by gamma and neutron radiation inside the 21-PWR (pressurized-water reactor) waste package (WP). The scope of this calculation is for the spent nuclear fuel (SNF) decay time period between ten to one million years.

The results of this calculation will be used to evaluate nitric acid corrosion of fuel cladding from radiolysis in the 21-PWR WP. The 21-PWR WP is a part of the Monitored Geologic Repository (MGR) Uncanistered SNF Disposal Container System. The MGR Uncanistered SNF Disposal Container System is classified as Quality Level 1 (CRWMS M&O [Civilian Radioactive Waste Management and Operator Contractor] 1999a, page 7). Therefore, this calculation is subject to the requirements of the Quality Assurance Requirements and Description (DOE [U.S. Department of Energy 2002]). The performance of the calculation and development of this document are carried out in accordance with AP-3.12Q, *Design Calculations and Analyses*. This calculation is associated with the total system performance assessment (TSPA) of which the integrity of the spent fuel cladding is to be evaluated.

The calculation method, input description, and results from this calculation are given in the following sections. The information provided by the sketch attached to this calculation is that of the potential design for the type of WP considered in this calculation.

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## 2. METHOD

The integrity of the cladding material of PWR SNF is one of the issues of the TSPA for the site recommendation (SR) study. It is recognized that chemical species produced by ionizing radiation (radiolysis) can have deleterious effect on the fuel cladding by nitric acid corrosion. Nitric acid is produced in an irradiated air-water vapor system when the hydroxyl radicals, generated from water vapor, convert nitrogen dioxides, formed by radiolytic reaction between nitrogen and oxygen, to nitric acids. Prior to waste package failure, radiolysis effect is insignificant because of the absence of water and air in the waste package. Therefore, it is only after waste package failure that radiation-enhanced corrosion of fuel cladding can occur. For this calculation the 21-PWR WP is assumed to have failed and air and water have entered the WP (Assumption 3.1).

Although radiolytic production of individual chemical species depends on the radiation environment, the chemical components present, and the physical conditions of the environment (temperature, pressure, and relative humidity), the yield of a chemical species is characterized by a single parameter, G value (Reed and Van Konyneburg, 1991). The G value represents the number of molecules of a chemical species produced per 100 eV of radiation energy absorbed in the ambient environment; it has the dimensions of (molecules/100 eV).

In this calculation, it is assumed that the 21-PWR WP has been breached, and air and water are present inside the WP (Assumption 3.1). For the failed waste package, the maximum amount of nitric acid production is determined by the total radiation energy absorbed by the moist air in the waste package. The 21-PWR WP is assumed to contain the average SNF (Assumption 3.2), and the energy deposition rates outside the fuel rods are calculated at 35 time steps within the time period between ten to one million years. Then, the rate of nitric acid production is determined for each time step based on the energy deposition and the G values of 1.5 for both neutron and gamma radiation (Assumptions 3.9 and 3.10). The average PWR SNF has the following characteristics: 4.0-wt% initial  $^{235}\text{U}$ , 48-GWd/MTU burnup, and 21-year decay time (Assumption 3.2).

Three-dimensional Monte Carlo calculations of the energy deposition rates and dose rates inside the 21-PWR WP are carried out using the MCNP program (Briesmeister 1997). The code collects information about events that occur during the random walks of particles in set of variables known as tallies. A series of these tallies have been specified in the MCNP input decks to obtain estimates for the energy deposition and particle flux in the moist air in the nine central assemblies. Information are collected for both gamma and neutron calculations using the "f6" and "f4" tallies that are defined as

- f6    energy deposition averaged over a cell in MeV/g
- f4    flux averaged average over a cell in particles/cm<sup>2</sup>.

The f6 results are converted to nitric production rates using the G values, the moist air density, and the total volume of the moist air. The f4 results are converted to dose rates using the ANSI/ANS-6.1.1-1977 flux-to-dose-rate conversion factors (Briesmeister 1997, pages H-5 and H-6).

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### 3. ASSUMPTIONS

- 3.1 It is assumed that the 21-PWR WP has failed and air and water are present in the WP. The rationale for this assumption is that air and water must be present for radiolysis to take place. This assumption is used in Section 5.
- 3.2 The PWR SNF having 4.0-wt% initial  $^{235}\text{U}$ , 48-GWd/MTU burnup, and 21-year decay time is assumed to be the SNF with average characteristics. The rationale for this assumption is that the source term for the SNF with these characteristics generates conservative (higher) radiation dose rates for an average PWR SNF (BSC [Bechtel SAIC Company] 2001, Assumption 3.3). This assumption is used in Section 5.
- 3.3 The chemical composition of the SNF is assumed to be the same as that of the fresh fuel. The rationale for this assumption is that small weight variations of the elements do not affect the accuracy of the gamma radiation energy absorption outside the SNF rods, as long as the total weight is maintained. Also, using fresh fuel for SNF results in conservative (higher) neutron energy absorption outside the SNF rods. This assumption is used in Section 5.
- 3.4 The non-fuel material compositions used in this calculation have elements with allowable ranges of weight percentages. For elements with weight percent range, the midpoint value is used, and the weight percent of the most abundant element is adjusted. The rationale for this assumption is that small weight variations for the affected elements do not affect the accuracy of dose results, as long as the total weight is maintained. This assumption is used throughout Section 5.
- 3.5 The basket assembly, the trunnion collar sleeves, the extended outer shell lid, the lifting features of the upper shell lids, and the bottom support ring in the 21-PWR WP are not represented in the geometry of the MCNP calculations. The rationale for this assumption is that excluding the basket assembly yields higher (conservative) radiation dose rate and the other components have negligible effects on the energy absorption in the WP. This assumption is used in Section 5.
- 3.6 For each PWR SNF assembly, the following items in the active fuel region are not included in the MCNP calculations: the guide tubes, instrument tube, incore-spacers, and grid supports. The rationale for this assumption is the same as that for Assumption 3.5. This assumption is used in Section 5.
- 3.7 For each PWR SNF assembly, the materials in the plenum region, top end-fitting region, and bottom end-fitting region are homogenized within the lateral boundaries of the assembly. The rationale for this assumption is that the reflective effects of these regions on energy deposition in the lattice cells are very small. Thus, homogenization of these regions should give the similar effect. This assumption is used in Section 5.

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- 3.8 It is assumed that the internal configuration of the 21-PWR WP remains intact for the entire time period of one million years. The rationale for this assumption is that the intact configuration leads to higher (conservative) radiolytic production of chemical species due to higher dose rate in the moist air in the WP. This assumption is used in Section 5.5.
- 3.9 It is assumed that the G value for nitrogen dioxide (or nitric acid) production by gamma radiation is 1.5 molecules/100 eV. The rationale for this assumption is that the G values between 0.5 to 1.5 molecules/100 eV have been observed in the long-term and lower dose rate gamma studies (Reed and Van Konynenburg 1991, p. 1399). Since this calculation is for long-term and low gamma irradiation conditions, G value of 1.5 is conservative by producing the highest of nitric acid. This assumption is used in Section 5.5.
- 3.10 It is assumed that the G value for nitric acid production by neutron radiation is 1.5 molecules/100 eV. The rationale for this assumption is that the G value of 1.5 molecules/100 eV absorbed by the nitrogen component of the gas phase in a air-water system has been reported (Reed and Van Konynenburg 1988, p. 401). Since the G value used in this calculation is per 100 eV energy absorbed by all components, using the G value of 1.5 is conservative for of nitric acid production. This assumption is used in Section 5.5.
- 3.11 It is assumed that the pressure in the failed 21-PWR WP is one atmosphere and the temperature in the WP is between 30° C and 90° C with relative humidity between 40% to 90%. The rationale for this assumption is that the WP has failed and exposed to the atmosphere and the temperature and relative humidity ranges correspond to those of the measured G values for nitrate production (Figure 3, Reed and Van Konyneburg, 1991). This assumption is used in Sections 5 and 6.
- 3.12 It is assumed that neutron spectra for PWR SNF at different decay times are identical. The rationale for this assumption is that neutron source in the commercial SNF is dominated by neutrons from spontaneous fission of actinides. Since spontaneous fission spectra of actinides are similar, the neutron spectra at different decay times should be nearly the same. This assumption is supported by Figure 4, which depicts the neutron source spectra of the average PWR SNF for time from one year to one million years. This assumption is used in Sections 5 and 6.

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#### 4. USE OF COMPUTER SOFTWARE AND MODELS

##### 4.1 COMPUTER SOFTWARE

###### 4.1.1 MCNP 4B2LV

The MCNP 4B2LV computer code (CRWMS M&O 1998a) was used to calculate gamma and neutron energy absorption and fluxes inside the WP. The software was qualified according to Office of Civilian Radioactive Waste Management (OCRWM) procedure AP-SI.1Q, *Software Management*.

The software specifications are as follows:

Program Name: MCNP

Version/revision number: Version 4B2

CSCI Number: 30033 V4B2LV (CRWMS M&O 1998a)

- Computer Type: Hewlett Packard (HP) 9000 Series Workstations
- Status/operating system: Qualified/HP-UX B.10.20
- Computer processing unit number: Installed on the CRWMS M&O workstation 'Bloom,' YMP Tag 700887.

The input and output files for the various MCNP calculations are contained on a compact disk-read only memory (CD-ROM) (Attachment III) with the files documented in Table 25. The calculation spreadsheets described in Sections 5 and 6 and included in Attachment III are such that an independent repetition of the calculations may be performed.

The MCNP software used was:

- (a) appropriate for the calculation of radiation dose rate and radiant energy deposition (MCNP is a Monte Carlo computer program designed for shielding and criticality calculations and for tracking neutron and gamma radiation)
- (b) used only within the range of validation (benchmark cases used in validation the code include a number of critical configurations involving UO<sub>2</sub> fuels and shielding configurations involving neutron and gamma particle transport (CRWMS M&O 1998b))
- (c) obtained from the Software Configuration Management in accordance with appropriate procedures.

###### 4.1.2 MICROSOFT EXCEL 97 SR-2

Title: Excel

Version/Revision Number: Microsoft® Excel 97 SR-2

- This software is installed on a personal computer running Microsoft Windows 95 with CRWMS M&O Tag number 116210.

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Microsoft Excel for Windows, Version 97 SR-2, was used in this calculation to translate the input data into the correct format and units using standard mathematical expressions and operations. It was also used to reformulate and display results graphically or in tabular form. The user-defined formulas, inputs, and results are documented in sufficient detail to allow an independent repetition of computations. Thus, Microsoft Excel is used only as a worksheet and not as a software routine. Microsoft Excel is an exempt software product according to OCRWM procedure AP-SI.1Q, Section 2.1.1, *Software Management*.

## 4.2 MODELS

Not used.

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## 5. CALCULATION

This section provides the radiation source terms, the geometry of the 21-PWR WP, and the chemical compositions of the materials used in MCNP calculations. The WP consists of the 21-PWR disposal container, 21 PWR SNF assemblies, and a basket assembly. Sketch SK-0175 REV 02 that is included in Attachment I describes the geometry and material specifications of the 21-PWR disposal container used in this calculation. So long as the internal configuration of the assemblies and basket does not change, minor changes in container wall thickness or other external dimensions will have only a negligible effect on the results of this calculation.

The number of digits in the values cited herein may be the result of a calculation or may reflect the input from another source; consequently, the number of digits should not be interpreted as an indication of accuracy.

### 5.1 21-PWR DISPOSAL CONTAINER

The 21-PWR disposal container primary consists of an inner reinforcement cylindrical shell, an outer shell, top and bottom inner shell lids, a bottom outer-shell lid, two top outer shell lids, upper and lower trunnion collar sleeves, and the basket assembly. The details of geometry and material specifications of the 21-PWR disposal container are described in sketch SK-0175 REV 02. The basket assembly, the trunnion collar sleeves, the extended outer shell lid, the extended lid reinforcement ring, the lifting features of the upper shell lids, and the bottom support ring are not represented in the MCNP calculations (Assumption 3.5). Table 1 presents the geometry and material specifications of the 21-PWR disposal container that are represented in the MCNP calculations.

The chemical compositions and the associated atom densities of SB-575 N06022 and SA-240 S31600 are given in Tables 2 and 3, respectively. For elements with weight percent range, the midpoint value is used, and the weight percent of the most abundant element is adjusted (Assumption 3.4).

The atom densities (AD) of the element/isotope contents, in atoms/b-cm are calculated according to the following equation (Harmon et al. 1994, p. B-2):

$$AD \text{ (atoms/b.cm)} = \frac{\text{material density (g/cm}^3\text{)} * \text{weight fraction}_{\text{isotope}} * N_A \text{ (atoms/mole)}}{10^{24} \text{ (b/cm}^2\text{)} * \text{atomic mass}_{\text{isotope}} \text{ (g/mole)}} \quad (\text{Eq.1})$$

In the above equation,  $N_A$  is the Avogadro constant, whose value is  $6.0221367 \times 10^{23}$  atoms per mole (Parrington et al. 1996, page 59).

Table 1. Geometry and Material Specifications for the 21-PWR Disposal Container

Component	Material	Characteristic	Dimension (mm)
Inner shell	SA-240 S31600	Thickness	50
Top and bottom Inner lids	SA-240 S31600	Thickness	95
Outer shell	SB-575 N06022	Thickness	20
Top and bottom outer lids	SB-575 N06022	Thickness	25

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Closure lid to outer lid gap	Void	Thickness	30
Closure lid	SB-575 N06022	Thickness	10
Inner lid to closure lid gap	Void	Thickness	30
Bottom outer lid to inner lid gap	Void	Thickness	70
Fuel basket tube	SA-516 K02700 <sup>a</sup>	Thickness	5
		Inner transverse dimension	226.4
Cavity	Void	Length	4,585
		Inner diameter	1,424

NOTE: <sup>a</sup>Represented as void in MCNP calculations (Assumption 3.5).

Table 2. Chemical Composition of SB-575 N06022

Element <sup>a</sup>	Weight Percent Range <sup>a</sup>	Weight Percent Used	Atomic Mass <sup>b</sup> (g/mole)	Atom Density (atoms/b•cm)
C	0.015 (max)	0.015	12.0107	6.5357E-05
Mn	0.50 (max)	0.5	54.93805	4.7629E-04
Si	0.08 (max)	0.08	28.0855	1.4907E-04
Cr	20.00-22.50	21.25	51.9961	2.1387E-02
Mo	12.5-14.5	13.5	95.94	7.3638E-03
Co	2.50 (max)	2.5	58.9332	2.2200E-03
W	2.5-3.5	3	183.84	8.5399E-04
V	0.35 (max)	0.35	50.9415	3.5956E-04
Fe	2.0-6.0	4	55.845	3.7484E-03
P	0.02 (max)	0.02	30.97376	3.3791E-05
S	0.02 (max)	0.02	32.066	3.2640E-05
Ni	Balance	54.765	58.6934	4.8830E-02
Total		100		8.6374E-02
		Densitv = 8.69 g/cm <sup>3</sup>		

SOURCE: <sup>a</sup>ASTM (American Society for Testing and Materials) B 575-97, Standard Specification for Low-Carbon Nickel-Molybdenum-Chromium, Low-Carbon Nickel-Chromium-Molybdenum, Low-Carbon Nickel-Chromium-Molybdenum-Copper and Low-Carbon Nickel-Chromium-Molybdenum-Tungsten Alloy Plate, Sheet, and Strip, p. 2.

<sup>b</sup>Parrington et al. 1996.

Table 3. Chemical Composition of SA-240 S31600

Element	Weight Percent Range <sup>a</sup>	Weight Percent Used	Atomic Mass <sup>c</sup> (g/mole)	Atom Density (atoms/b•cm)
Carbon	0.08 (max)	0.08	12.0107	3.2009E-04
Mn	2.00 (max)	2	54.93805	1.7495E-03
P	0.045 (max)	0.045	30.97376	6.9819E-05
S	0.030 (max)	0.03	32.066	4.4960E-05
Si	0.75 (max)	0.75	28.0855	1.2833E-03
Cr	16.00-18.00	17	51.9961	1.5712E-02
Ni	10.00-14.00	12	58.6934	9.8253E-03
Mo	2.00-3.00	2.5	95.94	1.2523E-03
N	0.10 (max)	0.1	14.00674	3.4310E-04
Fe	Balance	65.495	55.845	5.6361E-02
Total		100		8.6961E-02
		Density = 7.98 g/cm <sup>3</sup>		

SOURCE: <sup>a</sup>ASTM A 240/A 240M-97a, Standard Specification for Heat-Resisting Chromium and Chromium-Nickel Stainless Steel Plate, Sheet, and Strip for Pressure Vessels, p. 2.

<sup>b</sup>ASTM G 1-90, Standard Practice for Preparing, Cleaning, and Evaluating Corrosion Test Specimens, p. 7.

<sup>c</sup>Parrington et al. 1996.

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## 5.2 PWR SNF ASSEMBLIES

The PWR SNF assembly used in this calculation is a Babcock and Wilcox (B&W) 15x15 PWR SNF assembly. The mechanical design parameters for a B&W 15x15 PWR fuel assembly are obtained from (BSC 2001, Table 7). Table 4 presents these parameters.

Fresh fuel is assumed for the PWR SNF (Assumption 3.3). To be consistent with the values used in the source term calculations, the mass of uranium and uranium dioxide for each assembly are obtained from CRWMS M&O (1999b, Attachment I, p. I-1) instead of using the values in Table 4. The composition of fresh fuel is presented in Table 5. The composition of Zircalloy-4 is given in Table 6. For elements with weight percent range, the midpoint value is used, and the weight percent of the most abundant element is adjusted (Assumption 3.4).

In MCNP calculations, each PWR SNF assembly axially consists of four regions: the bottom end-fitting region, active fuel region, plenum region, and top end-fitting region. Except for the active fuel region, each assembly region is homogenized inside its volume (see Figures 1 through 3), resulting in a uniform distribution of the region contents inside each region volume (Assumption 3.7). For the active fuel region, the content of each cell in the assembly is represented explicitly, but the guide tubes, instrument tube, incore-spacers, and grid supports are not included (Assumption 3.6). The chemical compositions and the associated atomic densities of the three non-fuel regions of each PWR assembly are obtained from BSC (2001, Attachment I) and presented in Tables 7, 8, and 9.

Table 4. Mechanical Design Parameters of B&W 15x15 Mark B Fuel Assembly<sup>a</sup>

Design Component	Material	Zone	Characteristic	Value
Assembly	N/A (Not Applicable)	N/A	Width	8.536 in. (21.68144 cm)
			Length	165.625 in. (420.6875 cm)
Fuel pin	N/A	In core	Number per assembly	208
			Length	153.68 in. (390.3472 cm)
Fuel pellets	UO <sub>2</sub>	Active fuel	Mass/pin	5.58 lb. (2.53105 kg)
			Mass U/assembly	0.46363 metric tons
			Diameter	0.3686 in. (0.93624 cm)
			Stack length	141.8 in. (360.172 cm)
Cladding	Zircaloy-4	In core	Thickness	0.0265 in. (0.06731 cm)
			Fuel-clad gap	0.0042 in. (0.010668 cm)
Top nozzle	SS CF3M	Top	Mass/assembly	7.48 kg
Bottom nozzle	SS CF3M	Bottom	Mass/assembly	8.16 kg
Guide tube	Zircaloy-4	In core	Mass/assembly	8.0 kg
Instrument tube	Zircaloy-4	In core	Mass/assembly	0.64 kg
Spacer-plenum	Inconel-718	Plenum	Mass/assembly	1.04 kg
Spacer-bottom	Inconel-718	Bottom	Mass/assembly	1.3 kg
Spacer-incore	Inconel-718	In core	Mass/assembly	4.9 kg
Spring retainer	SS CF3M	Top	Mass/assembly	0.91 kg
Holding spring	Inconel-718	Top	Mass/assembly	1.8 kg
Upper end plug	SS 304	Top	Mass/assembly	0.06 kg
Upper nut	SS 304L	Top	Mass/assembly	0.51 kg
Lower nut	SS 304	Bottom	Mass/assembly	0.15 kg
Grid supports	Zircaloy-4	In core	Mass/assembly	0.64 kg
Plenum spring	SS 302	Plenum	Mass/assembly	0.042 lb. (0.0 kg)
Plenum region	N/A	N/A	Length <sup>b</sup>	30.1752 cm

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Bottom end-fitting	N/A	N/A	Length <sup>c</sup>	4 in. (10.16 cm)
Fuel pin pitch	N/A	N/A	Width <sup>d</sup>	1.44272 cm

SOURCE: <sup>a</sup>BSC 2001, Table 7.NOTES: <sup>b</sup>Calculated: fuel pin length – fuel pellet length = 390.3472 cm – 360.172 cm.<sup>c</sup>A bottom end-fitting region of 4-in. length provides conservative (higher) dose rates for bottom region of the WP.<sup>d</sup>Punatar 2001. Table 2-2.

Table 5. Chemical Composition of Fresh PWR Fuel

Isotope	Isotopic Mass <sup>a</sup> (g/mole)	Wt% of U <sup>b</sup>	Wt. Portion of UO <sub>2</sub>	Wt. Fraction of UO <sub>2</sub>	Atom Density (atoms/b•cm)
U-235	235.043922	4.0000	0.0400	0.0352	9.4426E-04
U-234	234.040945	0.0347	0.0003	0.0003	8.2334E-06
U-236	236.045561	0.0184	0.0002	0.0002	4.3252E-06
U-238	238.050785	95.9469	0.9595	0.8443	2.2364E-02
O	15.9994	N/A	0.1364	0.1200	4.7289E-02
Total	N/A	N/A	1.1364 <sup>c</sup>	1.0000	7.0609E-02
Density <sup>a</sup> = 10.47 g/cm <sup>3</sup>					

SOURCE: <sup>a</sup>Parrington et al. 1996.<sup>b</sup>BSC 2001, Table 1-5.<sup>c</sup>CRWMS M&O 1999b, Attachment I, p. I-1. Mass of UO<sub>2</sub>/mass of U = 539.771475 = 1.1364.<sup>d</sup>CRWMS M&O 1999b, Attachment I, p. I-1.

Table 6. Chemical Composition of Zircalloy-4

Element	Weight Percent Range <sup>a</sup>	Weight Percent Used	Atomic Mass <sup>c</sup> (g/mole)	Atom Density (atoms/b•cm)
Sn	1.20-1.70	1.45	118.71	4.8254E-04
Fe	0.18-0.24	0.21	55.845	1.4856E-04
Cr	0.07-0.13	0.115	51.9961	8.7374E-05
O	0.09-0.16	0.125	15.9994	3.0865E-04
Fe + Cr	0.28-0.37	0.325	N/A	N/A
Zr	Balance	98.1	91.224	4.2483E-02
Total	N/A	100	N/A	4.3510E-02
Density <sup>b</sup> = 6.56 g/cm <sup>3</sup>				

SOURCE: <sup>a</sup>ASTM B 811-90, Standard Specification for Wrought Zirconium Alloy Seamless Tubes for Nuclear Reactor Fuel Cladding, Table 2.<sup>b</sup>ASM International 1990, p. 666.<sup>c</sup>Parrington et al. 1996.Table 7. Chemical Composition of the Bottom End-Fitting Region<sup>a</sup>

Element/Isotope	Mass (g)	Nuclide Identification <sup>c</sup>	Atomic Mass <sup>b</sup> (g/mole)	Atom Density (atoms/b•cm)
C	3.608	6000.01p	12.0107	3.7877E-05
Mn	129.95	25055.01p	54.93805	2.9825E-04
P	0.2625	15031.01~	30.97376	1.0686E-06
S	0.24	16000.01p	32.066	9.4373E-07
Si	168.875	14000.01p	28.0855	7.5817E-04
Cr	1825.9	24000.01p	51.9961	4.4278E-03
Ni	1499.375	28000.01p	58.6934	3.2211E-03

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N	0.15	7014.01p	14.003074	1.3507E-06
Fe	5636.1865	26000.01p	55.845	1.2725E-02
Mo	243.65	42000.01p	95.94	3.2022E-04
Nb	66.625	41093.01p	92.90638	9.0422E-05
Ti	11.7	22000.01p	47.867	3.0820E-05
Al	6.5	13027.01p	26.98154	3.0376E-05
Co	13	27059.01p	58.9332	2.7814E-05
Cu	3.9	29000.01p	63.546	7.7385E-06
B	0.078	N/A	10.811	9.0972E-07
<sup>10</sup> B (19.9 at%)	N/A	5010.01p	N/A	1.8103E-07
<sup>11</sup> B (80.1 at%)	N/A	5011.01p	N/A	7.2869E-07
<b>Total</b>	9610	N/A	N/A	2.1981E-02

SOURCE: <sup>a</sup>BSC 2001, Attachment I, Table 1-3.<sup>b</sup>Parrington et al. 1996.<sup>c</sup>Briesmeister 1997, Appendix G.Originator: JS Date: 9/13/02Checker: GR Date: 9/16/02

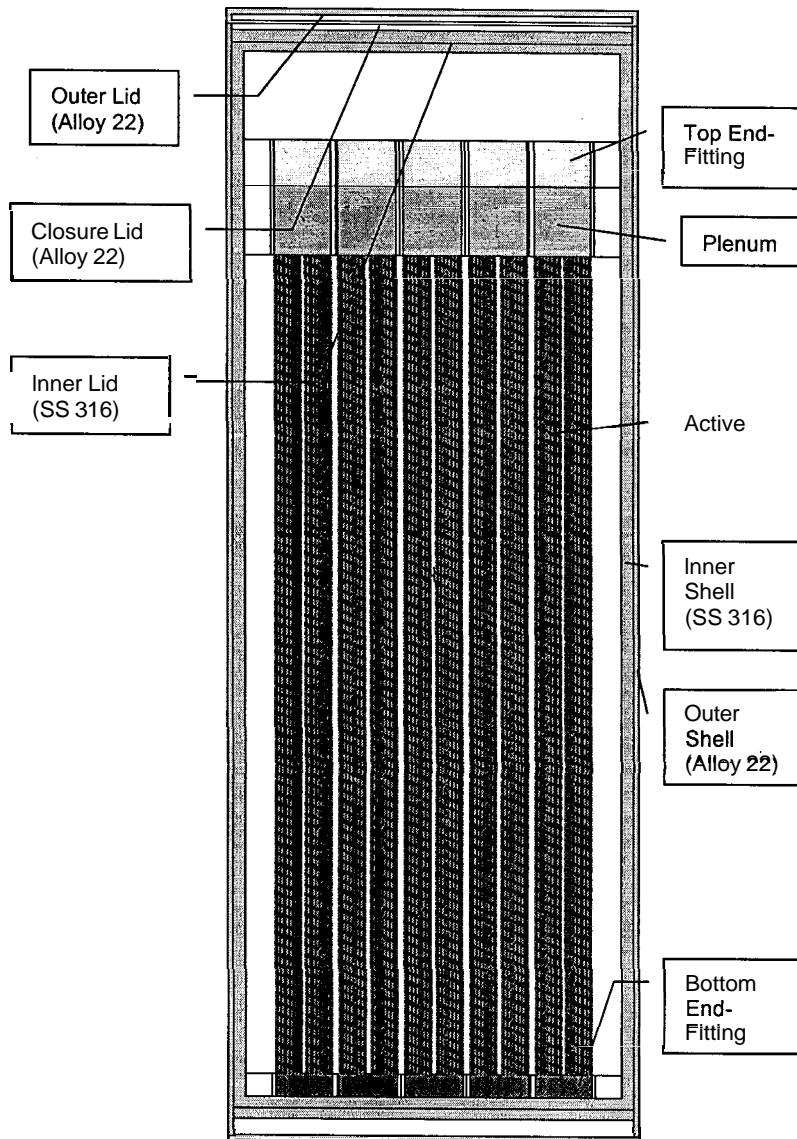


Figure 1. Source Region Representation in MCNP Calculations



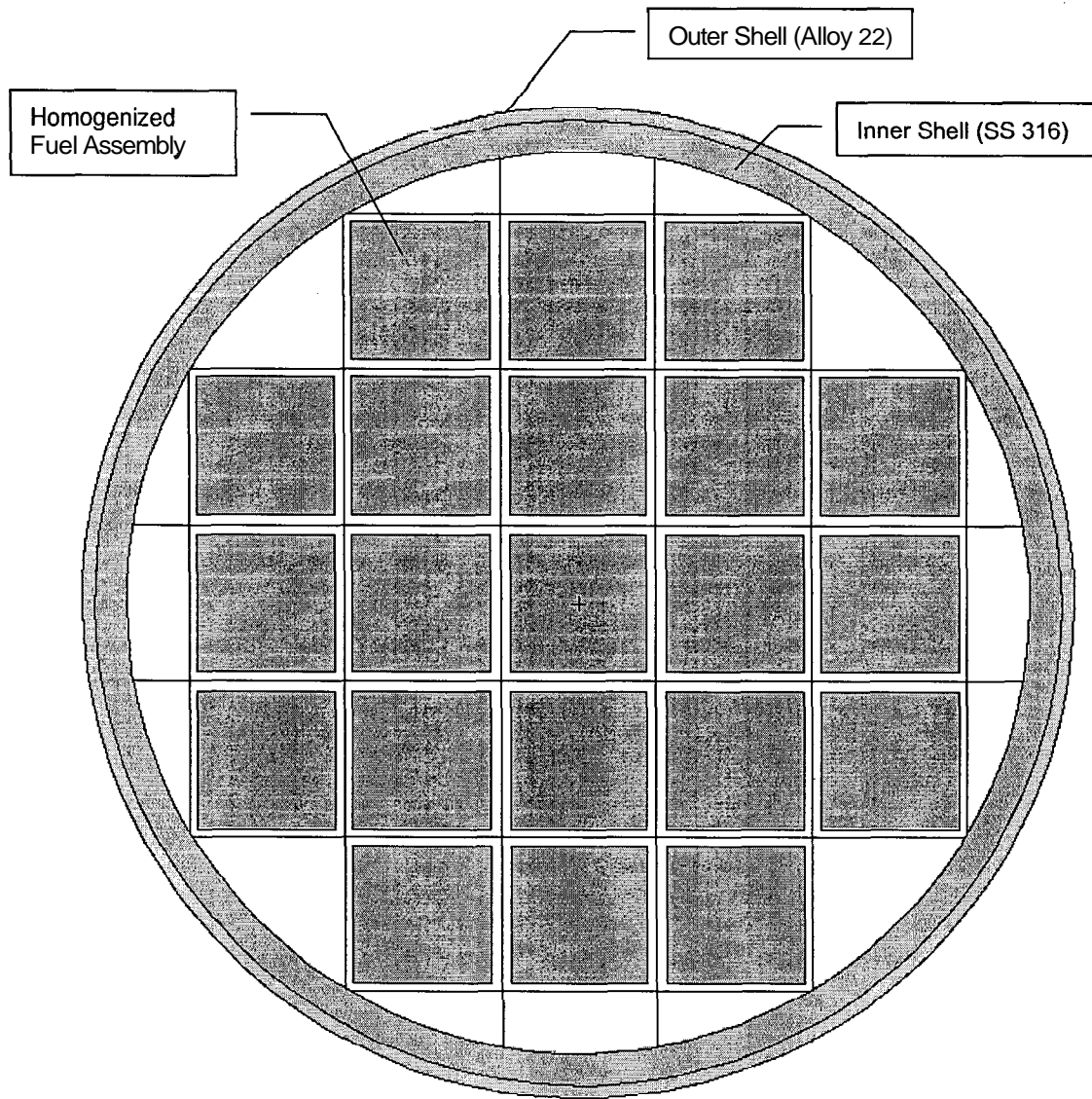


Figure 2. Lateral View of the WP Plenum Region in MCNP Calculations

Originator: JST Date: 9/13/02

Checker: GR Date: 9/16/02

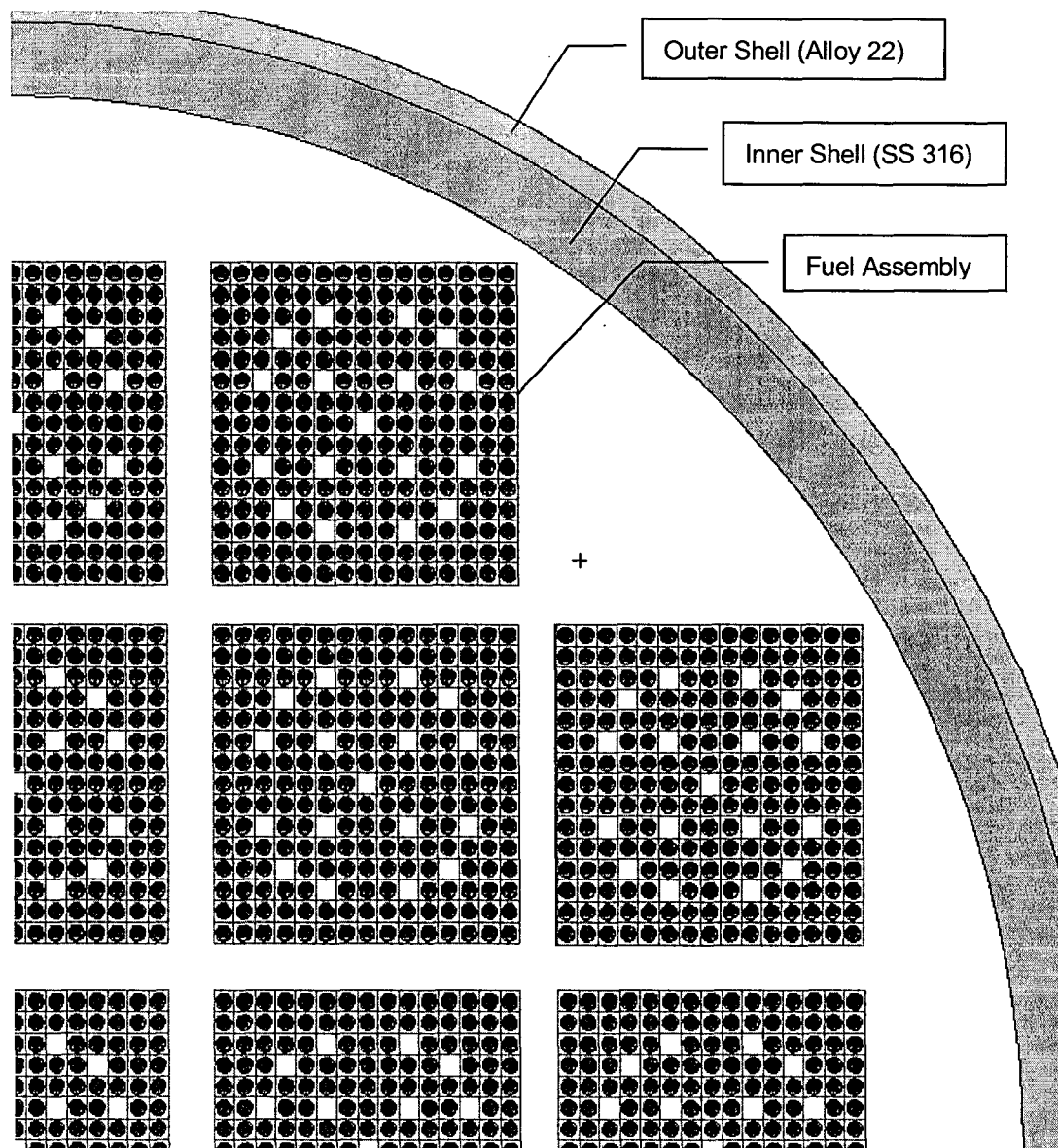


Figure 3. Partial Lateral View of the WP Active Fuel Region in MCNP Calculations

Originator: JSY Date: 9/13/02Checker: GR Date: 9/16/02

Table 8. Chemical Composition of the Plenum Region<sup>a</sup>

Element/Isotope	Mass (g)	Nuclide Identification <sup>c</sup>	Atomic Mass <sup>b</sup> (gmole)	Atom Density (atoms/b•cm)
Ni	537.3145	28000.01p	58.6934	3.8865E-04
Cr	212.05881	24000.01p	51.9961	1.7314E-04
Fe	218.6757	26000.01p	55.845	1.6624E-04
Nb	53.3	41093.01p	92.90638	2.4356E-05
Mo	31.72	42000.01p	95.94	1.4036E-05
Ti	9.36	22000.01p	47.867	8.3016E-06
Al	5.2	13027.01p	26.981538	8.1820E-06
Co	10.4	27059.01p	58.9332	7.4920E-06
Mn	4.021	25055.01p	54.938049	3.1073E-06
Si	3.782875	14000.01p	28.0855	5.7183E-06
Cu	3.12	29000.01p	63.546	2.0844E-06
C	0.860575	6000.01p	12.0107	3.0419E-06
S	0.161715	16000.01p	32.066	2.1411E-07
P	0.1645725	15031.01p	30.973761	2.2557E-07
B	0.0624	N/A	10.811	2.4504E-07
<sup>10</sup> B (19.9 at%)	N/A	5010.01p	N/A	4.8764E-08
<sup>11</sup> B (80.1 at%)	N/A	5011.01p	N/A	3.9060E-08
Sn	139.0715	50000.01p	118.71	4.9736E-05
O	11.988922	8016.01p	15.994915	3.1822E-05
Zr	9408.906	40000.01p	91.224	4.3788E-03
N	0.01905	7014.01p	14.003074	5.7741E-08
<b>Total</b>	N/A	N/A	N/A	5.2653E-03

SOURCE: <sup>a</sup>BSC 2001, Attachment I, Table I-10.<sup>b</sup>Parrington et al. 1996.<sup>c</sup>Briesmeister 1997, Appendix G.Table 9. Chemical Composition of the Top End-Fitting Region<sup>a</sup>

Element/Isotope	Mass (g)	Nuclide Identification <sup>c</sup>	Atomic Mass <sup>b</sup> (gmole)	Atom Density (atoms/b•cm)
C	4.158	6000.01p	12.0107	2.1977E-05
Mn	143.55	25055.01p	54.93805	1.6587E-04
P	0.5265	15031.01p	30.97376	1.0791E-06
S	0.441	16000.01p	32.066	8.7305E-07
Si	178.375	14000.01p	28.0855	4.0318E-04
Cr	2044.4	24000.01p	51.9961	2.4960E-03
Ni	1822.55	28000.01p	58.6934	1.9712E-03
N	0.57	7014.01p	14.003074	2.5840E-06
Fe	6159.8215	26000.01p	55.845	7.0021E-03
Mo	264.65	42000.01p	95.94	1.7511E-04
Nb	92.25	41093.01p	92.90638	6.3033E-05
Ti	16.2	22000.01p	47.867	2.1485E-05
Al	9	13027.01p	26.98154	2.1175E-05

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Co	18	27059.01p	58.9332	1.9389E-05
Cu	5.4	29000.01p	63.546	5.3945E-06
B	0.108	NIA	10.811	6.3417E-07
<sup>10</sup> B (19.9 at%)	N/A	5010.01p	N/A	1.2620E-07
<sup>11</sup> B (80.1 at%)	NIA	5011.01p	NIA	5.0797E-07
<b>Total</b>	10760	N/A	N/A	1.2371E-02

SOURCE: <sup>a</sup>BSC 2001, Attachment I, Table 1-13.<sup>b</sup>Parrington et al. 1996.<sup>c</sup>Briesmeister 1997, Appendix G.

### 5.3 RADIATION SOURCES

Neutron and gamma radiation sources for the time period between ten and one million years are obtained from CRWMS M&O (1999c, disk 1 of 5), and presented in Tables 10 and 11, respectively.

The neutron source spectra per assembly are presented in eight energy groups, starting from 20 MeV and descending to the thermal energy. There are no neutrons below energy of 0.1 MeV. The total neutron source intensities per WP are provided in the last column. The gamma source spectra per assembly are given in 18 energy groups, starting from 10 keV and ascending to 10 MeV. Finally, the total gamma source intensities in the WP are included in the last column of Table 11.

The neutron source spectra per assembly for the time period of one to one million years are depicted in Figure 4. It is noted that the spectra are nearly identical for the all time steps. Therefore, only one MCNP calculation is needed, and the results for all time steps can be obtained by scaling the tallies to the source intensity of each time step.

Table 10. Neutron Source Spectra (n/s) of the Average PWR SNF Assembly

Age (Years)	Upper Energy Boundary (MeV)								Total per WP
	2.00E+01	6.43E+00	3.00E+00	1.85E+00	1.40E+00	9.00E-01	4.00E-01	1.00E-01	
1	9.92E+06	1.16E+08	1.27E+08	7.02E+07	9.49E+07	1.04E+08	2.03E+07	0.00E+00	1.14E+10
2	8.88E+06	1.01E+08	1.11E+08	6.27E+07	8.50E+07	9.29E+07	1.82E+07	0.00E+00	1.01E+10
3	8.40E+06	9.52E+07	1.05E+08	5.93E+07	8.05E+07	8.79E+07	1.72E+07	0.00E+00	9.52E+09
4	8.06E+06	9.12E+07	1.01E+08	5.69E+07	7.72E+07	8.43E+07	1.65E+07	0.00E+00	9.14E+09
5	7.75E+06	8.77E+07	9.67E+07	5.47E+07	7.42E+07	8.10E+07	1.59E+07	0.00E+00	8.78E+09
7	7.18E+06	8.13E+07	8.97E+07	5.07E+07	6.88E+07	7.51E+07	1.47E+07	0.00E+00	8.14E+09
10	6.40E+06	7.27E+07	8.04E+07	4.54E+07	6.14E+07	6.70E+07	1.31E+07	0.00E+00	7.27E+09
15	5.30E+06	6.04E+07	6.70E+07	3.77E+07	5.09E+07	5.55E+07	1.09E+07	0.00E+00	6.04E+09
20	4.40E+06	5.02E+07	5.60E+07	3.14E+07	4.23E+07	4.60E+07	9.01E+06	0.00E+00	5.03E+09
25	3.65E+06	4.18E+07	4.69E+07	2.61E+07	3.52E+07	3.82E+07	7.48E+06	0.00E+00	4.19E+09
30	3.03E+06	3.49E+07	3.93E+07	2.18E+07	2.93E+07	3.18E+07	6.22E+06	0.00E+00	3.49E+09
35	2.52E+06	2.92E+07	3.31E+07	1.82E+07	2.44E+07	2.64E+07	5.17E+06	0.00E+00	2.92E+09
40	2.10E+06	2.44E+07	2.80E+07	1.53E+07	2.04E+07	2.20E+07	4.31E+06	0.00E+00	2.45E+09
45	1.75E+06	2.05E+07	2.37E+07	1.29E+07	1.71E+07	1.84E+07	3.60E+06	0.00E+00	2.06E+09
50	1.46E+06	1.73E+07	2.02E+07	1.09E+07	1.43E+07	1.54E+07	3.01E+06	0.00E+00	1.73E+09
60	1.03E+06	1.24E+07	1.48E+07	7.80E+06	1.02E+07	1.09E+07	2.13E+06	0.00E+00	1.24E+09
70	7.36E+05	9.05E+06	1.11E+07	5.72E+06	7.34E+06	7.78E+06	1.52E+06	0.00E+00	9.08E+08
80	5.34E+05	6.76E+06	8.62E+06	4.29E+06	5.41E+06	5.67E+06	1.11E+06	0.00E+00	6.80E+08
100	3.03E+05	4.10E+06	5.66E+06	2.64E+06	3.18E+06	3.25E+06	6.33E+05	0.00E+00	4.15E+08
150	1.30E+05	2.06E+06	3.27E+06	1.36E+06	1.49E+06	1.43E+06	2.78E+05	0.00E+00	2.10E+08
200	1.04E+05	1.68E+06	2.72E+06	1.11E+06	1.21E+06	1.15E+06	2.22E+05	0.00E+00	1.72E+08

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250	9.89E+04	1.56E+06	2.50E+06	1.03E+06	1.13E+06	1.09E+06	2.11E+05	0.00E+00	1.60E+08
300	9.79E+04	1.50E+06	2.35E+06	9.84E+05	1.10E+06	1.07E+06	2.08E+05	0.00E+00	1.54E+08
350	9.73E+04	1.45E+06	2.23E+06	9.47E+05	1.07E+06	1.06E+06	2.06E+05	0.00E+00	1.48E+08
400	9.67E+04	1.41E+06	2.14E+06	9.17E+05	1.05E+06	1.04E+06	2.04E+05	0.00E+00	1.44E+08
500	9.54E+04	1.35E+06	1.98E+06	8.70E+05	1.01E+06	1.02E+06	2.01E+05	0.00E+00	1.37E+08
700	9.31E+04	1.26E+06	1.76E+06	8.01E+05	9.61E+05	9.92E+05	1.95E+05	0.00E+00	1.27E+08
1000	9.00E+04	1.16E+06	1.54E+06	7.30E+05	9.03E+05	9.51E+05	1.87E+05	0.00E+00	1.17E+08
1500	8.51E+04	1.06E+06	1.31E+06	6.50E+05	8.31E+05	8.93E+05	1.76E+05	0.00E+00	1.05E+08
2000	8.06E+04	9.92E+05	1.18E+06	5.96E+05	7.76E+05	8.42E+05	1.66E+05	0.00E+00	9.73E+07
2500	7.63E+04	9.32E+05	1.09E+06	5.58E+05	7.31E+05	7.96E+05	1.57E+05	0.00E+00	9.11E+07
3000	7.24E+04	8.80E+05	1.03E+06	5.26E+05	6.91E+05	7.54E+05	1.49E+05	0.00E+00	8.62E+07
3500	6.86E+04	8.34E+05	9.70E+05	4.98E+05	6.55E+05	7.15E+05	1.41E+05	0.00E+00	8.15E+07
4000	6.52E+04	7.91E+05	9.20E+05	4.73E+05	6.22E+05	6.79E+05	1.34E+05	0.00E+00	7.74E+07
4500	6.19E+04	7.53E+05	8.76E+05	4.50E+05	5.91E+05	6.45E+05	1.27E+05	0.00E+00	7.36E+07
5000	5.89E+04	7.16E+05	8.34E+05	4.28E+05	5.62E+05	6.14E+05	1.21E+05	0.00E+00	7.00E+07
5500	5.61E+04	6.83E+05	7.96E+05	4.08E+05	5.36E+05	5.84E+05	1.15E+05	0.00E+00	6.67E+07
6000	5.34E+04	6.51E+05	7.59E+05	3.89E+05	5.11E+05	5.57E+05	1.10E+05	0.00E+00	6.36E+07
6500	5.06E+04	6.22E+05	7.24E+05	3.73E+05	4.89E+05	5.30E+05	1.04E+05	0.00E+00	6.07E+07
7000	4.80E+04	5.94E+05	6.91E+05	3.58E+05	4.69E+05	5.04E+05	9.87E+04	0.00E+00	5.80E+07
7500	4.58E+04	5.65E+05	6.61E+05	3.44E+05	4.49E+05	4.82E+05	9.42E+04	0.00E+00	5.55E+07
8000	4.37E+04	5.39E+05	6.33E+05	3.31E+05	4.30E+05	4.61E+05	9.01E+04	0.00E+00	5.31E+07
8500	4.19E+04	5.16E+05	6.07E+05	3.17E+05	4.12E+05	4.41E+05	8.63E+04	0.00E+00	5.08E+07
9000	4.01E+04	4.95E+05	5.83E+05	3.04E+05	3.95E+05	4.23E+05	8.27E+04	0.00E+00	4.88E+07
9500	3.85E+04	4.75E+05	5.60E+05	2.92E+05	3.79E+05	4.05E+05	7.93E+04	0.00E+00	4.68E+07
10000	3.69E+04	4.57E+05	5.38E+05	2.80E+05	3.64E+05	3.89E+05	7.61E+04	0.00E+00	4.50E+07
15000	2.60E+04	3.23E+05	3.83E+05	1.98E+05	2.57E+05	2.74E+05	5.36E+04	0.00E+00	3.18E+07
20000	2.02E+04	2.51E+05	2.98E+05	1.54E+05	2.00E+05	2.13E+05	4.17E+04	0.00E+00	2.47E+07
25000	1.71E+04	2.12E+05	2.50E+05	1.30E+05	1.69E+05	1.80E+05	3.53E+04	0.00E+00	2.09E+07
30000	1.54E+04	1.89E+05	2.21E+05	1.16E+05	1.52E+05	1.62E+05	3.18E+04	0.00E+00	1.86E+07
35000	1.45E+04	1.75E+05	2.04E+05	1.08E+05	1.42E+05	1.52E+05	2.97E+04	0.00E+00	1.73E+07
40000	1.39E+04	1.67E+05	1.93E+05	1.03E+05	1.35E+05	1.46E+05	2.85E+04	0.00E+00	1.65E+07
45000	1.35E+04	1.61E+05	1.85E+05	9.94E+04	1.31E+05	1.42E+05	2.77E+04	0.00E+00	1.60E+07
50000	1.32E+04	1.57E+05	1.79E+05	9.69E+04	1.29E+05	1.39E+05	2.72E+04	0.00E+00	1.56E+07
55000	1.30E+04	1.54E+05	1.75E+05	9.50E+04	1.26E+05	1.37E+05	2.68E+04	0.00E+00	1.53E+07
60000	1.29E+04	1.51E+05	1.71E+05	9.34E+04	1.25E+05	1.35E+05	2.64E+04	0.00E+00	1.50E+07
65000	1.27E+04	1.49E+05	1.68E+05	9.20E+04	1.23E+05	1.34E+05	2.61E+04	0.00E+00	1.48E+07
70000	1.26E+04	1.47E+05	1.65E+05	9.08E+04	1.22E+05	1.32E+05	2.59E+04	0.00E+00	1.46E+07
75000	1.25E+04	1.45E+05	1.62E+05	8.97E+04	1.20E+05	1.31E+05	2.56E+04	0.00E+00	1.44E+07
80000	1.24E+04	1.43E+05	1.60E+05	8.86E+04	1.19E+05	1.30E+05	2.53E+04	0.00E+00	1.42E+07
85000	1.22E+04	1.41E+05	1.58E+05	8.76E+04	1.18E+05	1.28E+05	2.51E+04	0.00E+00	1.41E+07
90000	1.21E+04	1.40E+05	1.56E+05	8.67E+04	1.17E+05	1.27E+05	2.49E+04	0.00E+00	1.39E+07
95000	1.20E+04	1.38E+05	1.54E+05	8.57E+04	1.16E+05	1.26E+05	2.46E+04	0.00E+00	1.38E+07
100000	1.19E+04	1.37E+05	1.52E+05	8.49E+04	1.14E+05	1.25E+05	2.44E+04	0.00E+00	1.36E+07
150000	1.09E+04	1.24E+05	1.37E+05	7.70E+04	1.04E+05	1.14E+05	2.22E+04	0.00E+00	1.24E+07
200000	9.90E+03	1.13E+05	1.25E+05	7.03E+04	9.50E+04	1.04E+05	2.03E+04	0.00E+00	1.13E+07
250000	9.03E+03	1.04E+05	1.15E+05	6.43E+04	8.67E+04	9.45E+04	1.85E+04	0.00E+00	1.03E+07
300000	8.23E+03	9.46E+04	1.05E+05	5.88E+04	7.92E+04	8.62E+04	1.69E+04	0.00E+00	9.43E+06
350000	7.51E+03	8.65E+04	9.64E+04	5.37E+04	7.23E+04	7.87E+04	1.54E+04	0.00E+00	8.62E+06
400000	6.85E+03	7.91E+04	8.84E+04	4.91E+04	6.60E+04	7.18E+04	1.41E+04	0.00E+00	7.88E+06
450000	6.25E+03	7.24E+04	8.10E+04	4.49E+04	6.03E+04	6.55E+04	1.28E+04	0.00E+00	7.21E+06
500000	5.71E+03	6.62E+04	7.42E+04	4.10E+04	5.50E+04	5.98E+04	1.17E+04	0.00E+00	6.59E+06
550000	5.21E+03	6.06E+04	6.81E+04	3.75E+04	5.03E+04	5.46E+04	1.07E+04	0.00E+00	6.03E+06
600000	4.75E+03	5.55E+04	6.25E+04	3.44E+04	4.60E+04	4.98E+04	9.76E+03	0.00E+00	5.52E+06
650000	4.34E+03	5.08E+04	5.73E+04	3.14E+04	4.20E+04	4.55E+04	8.91E+03	0.00E+00	5.05E+06
700000	3.97E+03	4.65E+04	5.26E+04	2.88E+04	3.84E+04	4.16E+04	8.14E+03	0.00E+00	4.62E+06
750000	3.62E+03	4.26E+04	4.84E+04	2.64E+04	3.51E+04	3.80E+04	7.44E+03	0.00E+00	4.23E+06
800000	3.31E+03	3.91E+04	4.45E+04	2.42E+04	3.21E+04	3.47E+04	6.80E+03	0.00E+00	3.88E+06
850000	3.03E+03	3.59E+04	4.09E+04	2.22E+04	2.94E+04	3.18E+04	6.22E+03	0.00E+00	3.56E+06

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900000	2.77E+03	3.29E+04	3.77E+04	2.03E+04	2.69E+04	2.91E+04	5.69E+03	0.00E+00	3.26E+06
950000	2.53E+03	3.02E+04	3.47E+04	1.87E+04	2.47E+04	2.66E+04	5.20E+03	0.00E+00	3.00E+06
1000000	2.32E+03	2.78E+04	3.20E+04	1.71E+04	2.26E+04	2.43E+04	4.76E+03	0.00E+00	2.75E+06

SOURCE: CRWMS M&amp;O 1999c, disk 1 of 5, file PWR.neutron.

Neutron Spectra for PWR SNF - 4%48GWd - over 1 Million Years

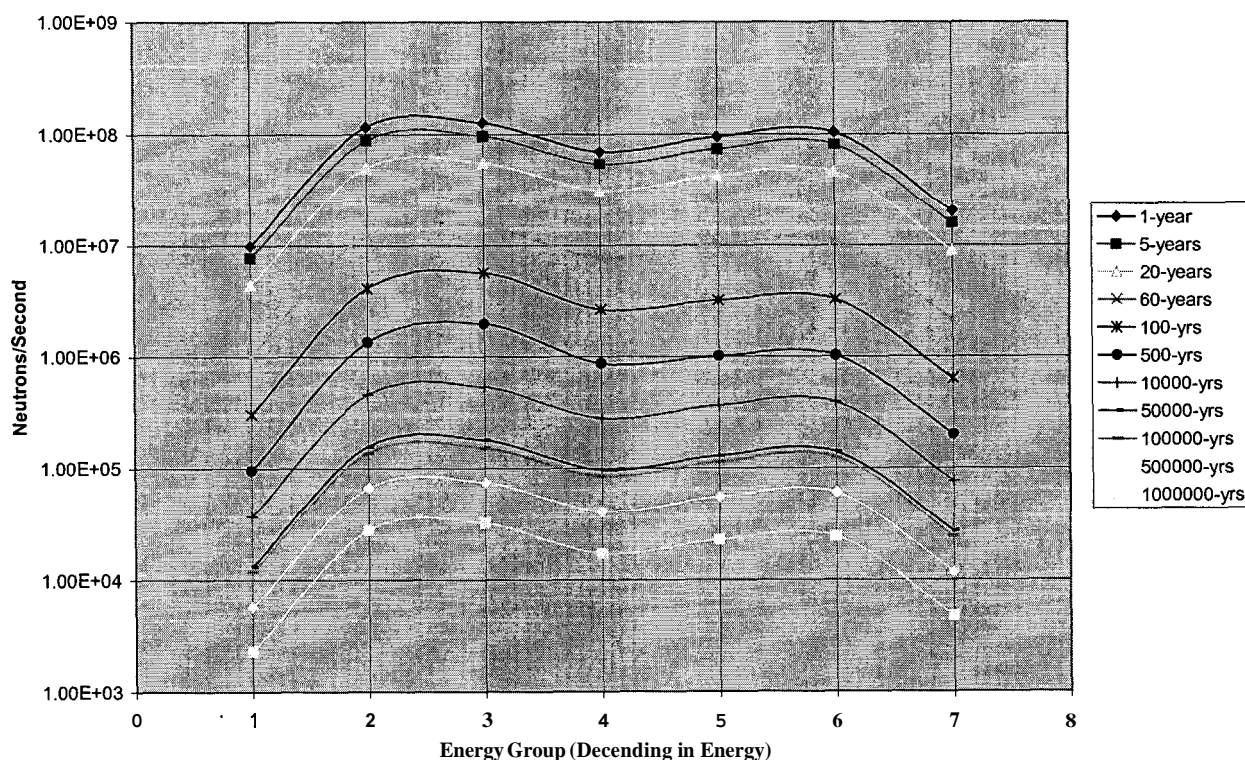


Figure 4. Neutron Source Spectra of the Average PWR SNF Assembly

Table 11. Gamma Source Spectra (p/s) of the Average PWR SNF Assembly

Age (Years)	Upper Energy Boundary (MeV)						
	5.00E-02	1.00E-01	2.00E-01	3.00E-01	4.00E-01	6.00E-01	8.00E-01
10	1.02E+15	2.83E+14	2.07E+14	6.07E+13	3.95E+13	1.69E+14	1.90E+15
15	8.70E+14	2.47E+14	1.71E+14	5.15E+13	3.39E+13	5.28E+13	1.59E+15
20	7.61E+14	2.21E+14	1.46E+14	4.46E+13	2.98E+13	2.79E+13	1.39E+15
25	6.70E+14	1.99E+14	1.26E+14	3.89E+13	2.63E+13	2.05E+13	1.24E+15
35	5.24E+14	1.63E+14	9.57E+13	2.98E+13	2.05E+13	1.46E+13	9.78E+14
50	3.66E+14	1.22E+14	6.47E+13	2.04E+13	1.42E+13	9.67E+12	6.90E+14
70	2.30E+14	8.58E+13	3.93E+13	1.25E+13	8.66E+12	5.79E+12	4.34E+14
100	1.17E+14	5.49E+13	1.90E+13	6.08E+12	4.16E+12	2.75E+12	2.17E+14
150	4.25E+13	3.33E+13	5.85E+12	1.95E+12	1.24E+12	8.11E+11	6.84E+13
200	1.94E+13	2.57E+13	2.00E+12	7.40E+11	3.87E+11	2.46E+11	2.16E+13
250	1.19E+13	2.22E+13	8.61E+11	3.85E+11	1.38E+11	8.11E+10	6.83E+12
350	7.80E+12	1.84E+13	4.22E+11	2.48E+11	4.36E+10	1.88E+10	7.27E+11
500	5.99E+12	1.46E+13	3.72E+11	2.32E+11	3.54E+10	1.31E+10	7.63E+10
700	4.57E+12	1.07E+13	3.61E+11	2.28E+11	3.56E+10	1.29E+10	5.52E+10
1000	3.21E+12	6.87E+12	3.49E+11	2.21E+11	3.57E+10	1.29E+10	5.46E+10
1500	1.98E+12	3.39E+12	3.31E+11	2.11E+11	3.54E+10	1.29E+10	5.40E+10

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2000	1.41E+12	1.82E+12	3.15E+11	2.02E+11	3.47E+10	1.29E+10	5.35E+10
2500	1.14E+12	1.10E+12	3.00E+11	1.93E+11	3.38E+10	1.28E+10	5.31E+10
3500	9.04E+11	6.02E+11	2.74E+11	1.76E+11	3.21E+10	1.27E+10	5.23E+10
5000	7.66E+11	4.38E+11	2.39E+11	1.53E+11	2.98E+10	1.26E+10	5.11E+10
7000	6.47E+11	3.61E+11	1.99E+11	1.28E+11	2.71E+10	1.25E+10	4.97E+10
10000	5.11E+11	2.80E+11	1.51E+11	9.76E+10	2.42E+10	1.23E+10	4.79E+10
15000	3.55E+11	1.88E+11	9.72E+10	6.31E+10	2.10E+10	1.21E+10	4.52E+10
20000	2.58E+11	1.31E+11	6.34E+10	4.18E+10	1.93E+10	1.19E+10	4.29E+10
25000	1.97E+11	9.58E+10	4.24E+10	2.88E+10	1.85E+10	1.18E+10	4.10E+10
35000	1.33E+11	6.06E+10	2.14E+10	1.62E+10	1.83E+10	1.15E+10	3.80E+10
50000	9.76E+10	4.50E+10	1.18E+10	1.13E+10	1.94E+10	1.12E+10	3.49E+10
70000	8.27E+10	4.27E+10	1.01E+10	1.15E+10	2.11E+10	1.08E+10	3.24E+10
100000	7.69E+10	4.48E+10	1.10E+10	1.34E+10	2.31E+10	1.03E+10	3.01E+10
150000	7.68E+10	4.81E+10	1.26E+10	1.57E+10	2.49E+10	9.78E+09	2.76E+10
200000	7.79E+10	4.96E+10	1.35E+10	1.66E+10	2.52E+10	9.35E+09	2.50E+10
250000	7.81E+10	5.01E+10	1.39E+10	1.67E+10	2.47E+10	9.04E+09	2.24E+10
350000	7.64E+10	4.94E+10	1.41E+10	1.57E+10	2.27E+10	8.62E+09	1.77E+10
500000	7.14E+10	4.69E+10	1.35E+10	1.36E+10	1.94E+10	8.21E+09	1.25E+10
700000	6.46E+10	4.31E+10	1.25E+10	1.13E+10	1.59E+10	7.75E+09	8.22E+09
1000000	5.65E+10	3.82E+10	1.11E+10	9.00E+09	1.27E+10	7.07E+09	5.14E+09

Table 11. Gamma Source Spectra (p/s) of the Average PWR SNF Assembly (Continued)

Age (Years)	Upper Energy Boundary (MeV)						
	1.00E+00	1.33E+00	1.66E+00	2.00E+00	2.50E+00	3.00E+00	4.00E+00
10	8.96E+13	1.55E+14	3.62E+13	1.30E+11	4.73E+10	3.05E+09	3.82E+08
15	3.18E+13	8.64E+13	1.79E+13	8.84E+10	5.70E+09	4.02E+08	4.03E+07
20	1.67E+13	4.98E+13	9.42E+12	7.68E+10	4.09E+09	3.05E+08	2.43E+07
25	1.10E+13	2.95E+13	5.13E+12	6.75E+10	3.53E+09	2.88E+08	1.98E+07
35	5.88E+12	1.12E+13	1.68E+12	5.24E+10	2.70E+09	2.57E+08	1.37E+07
50	2.79E+12	3.33E+12	4.39E+11	3.60E+10	1.85E+09	2.17E+08	7.99E+06
70	1.31E+12	1.03E+12	1.51E+11	2.19E+10	1.13E+09	1.75E+08	4.04E+06
100	5.67E+11	3.40E+11	6.02E+10	1.05E+10	5.38E+08	1.29E+08	1.69E+06
150	1.79E+11	9.10E+10	1.71E+10	3.06E+09	1.58E+08	7.77E+07	7.48E+05
200	7.04E+10	2.68E+10	5.03E+09	8.97E+08	4.70E+07	4.75E+07	6.01E+05
250	3.88E+10	8.15E+09	1.51E+09	2.67E+08	1.48E+07	2.91E+07	5.72E+05
350	2.67E+10	1.11E+09	1.84E+08	2.97E+07	2.97E+06	1.12E+07	5.56E+05
500	2.55E+10	4.54E+08	6.40E+07	1.03E+07	2.72E+06	3.02E+06	5.45E+05
700	2.53E+10	4.28E+08	6.34E+07	1.42E+07	4.38E+06	9.79E+05	5.37E+05
1000	2.50E+10	4.31E+08	6.89E+07	2.36E+07	7.97E+06	7.07E+05	5.32E+05
1500	2.46E+10	4.54E+08	8.21E+07	4.57E+07	1.64E+07	7.98E+05	5.36E+05
2000	2.42E+10	4.87E+08	9.92E+07	7.43E+07	2.73E+07	9.50E+05	5.53E+05
2500	2.38E+10	5.25E+08	1.20E+08	1.08E+08	4.02E+07	1.14E+06	5.79E+05
3500	2.31E+10	6.16E+08	1.67E+08	1.87E+08	7.04E+07	1.59E+06	6.53E+05
5000	2.20E+10	7.74E+08	2.50E+08	3.24E+08	1.23E+08	2.39E+06	7.99E+05
7000	2.06E+10	1.00E+09	3.71E+08	5.23E+08	1.99E+08	3.58E+06	1.03E+06
10000	1.88E+10	1.36E+09	5.58E+08	8.30E+08	3.16E+08	5.44E+06	1.41E+06
15000	1.61E+10	1.94E+09	8.65E+08	1.33E+09	5.06E+08	8.51E+06	2.07E+06
20000	1.38E+10	2.48E+09	1.16E+09	1.80E+09	6.87E+08	1.14E+07	2.72E+06
25000	1.20E+10	3.00E+09	1.43E+09	2.25E+09	8.58E+08	1.42E+07	3.35E+06
35000	9.05E+09	3.94E+09	1.94E+09	3.06E+09	1.17E+09	1.93E+07	4.51E+06
50000	6.23E+09	5.15E+09	2.60E+09	4.10E+09	1.57E+09	2.58E+07	6.00E+06
70000	4.22E+09	6.42E+09	3.30E+09	5.20E+09	1.99E+09	3.27E+07	7.58E+06
100000	2.99E+09	7.75E+09	4.05E+09	6.36E+09	2.43E+09	4.00E+07	9.24E+06
150000	2.52E+09	9.00E+09	4.78E+09	7.46E+09	2.85E+09	4.69E+07	1.08E+07
200000	2.40E+09	9.25E+09	4.98E+09	7.69E+09	2.94E+09	4.83E+07	1.11E+07
250000	2.28E+09	9.01E+09	4.92E+09	7.51E+09	2.87E+09	4.72E+07	1.09E+07

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350000	1.98E+09	7.91E+09	4.43E+09	6.59E+09	2.52E+09	4.14E+07	9.55E+06
500000	1.52E+09	6.05E+09	3.54E+09	5.03E+09	1.92E+09	3.16E+07	7.28E+06
700000	1.08E+09	4.20E+09	2.61E+09	3.47E+09	1.32E+09	2.19E+07	5.02E+06
1000000	7.30E+08	2.70E+09	1.83E+09	2.20E+09	8.40E+08	1.40E+07	3.19E+06

Table 11. Gamma Source Spectra (p/s) of the Average PWR SNF Assembly (Continued)

Age (Years)	Upper Energy Bound (MeV)				Total per WP
	5.00E+00	6.50E+00	8.00E+00	1.00E+01	
10	1.17E+07	4.71E+06	9.24E+05	1.96E+05	8.32E+16
15	9.72E+06	3.90E+06	7.65E+05	1.62E+05	6.62E+16
20	8.06E+06	3.24E+06	6.35E+05	1.35E+05	5.66E+16
25	6.69E+06	2.69E+06	5.27E+05	1.12E+05	4.97E+16
35	4.63E+06	1.86E+06	3.64E+05	7.73E+04	3.87E+16
50	2.70E+06	1.08E+06	2.12E+05	4.50E+04	2.72E+16
70	1.36E+06	5.46E+05	1.07E+05	2.27E+04	1.72E+16
100	5.68E+05	2.27E+05	4.44E+04	9.42E+03	8.86E+15
150	2.51E+05	9.99E+04	1.95E+04	4.12E+03	3.24E+15
200	2.01E+05	8.00E+04	1.56E+04	3.29E+03	1.47E+15
250	1.91E+05	7.62E+04	1.49E+04	3.14E+03	8.91E+14
350	1.86E+05	7.41E+04	1.45E+04	3.06E+03	5.81E+14
500	1.81E+05	7.23E+04	1.41E+04	2.99E+03	4.48E+14
700	1.76E+05	7.05E+04	1.38E+04	2.92E+03	3.36E+14
1000	1.70E+05	6.81E+04	1.33E+04	2.82E+03	2.26E+14
1500	1.61E+05	6.45E+04	1.26E+04	2.68E+03	1.27E+14
2000	1.53E+05	6.12E+04	1.20E+04	2.54E+03	8.13E+13
2500	1.45E+05	5.81E+04	1.14E+04	2.42E+03	6.00E+13
3500	1.31E+05	5.27E+04	1.03E+04	2.19E+03	4.36E+13
5000	1.14E+05	4.57E+04	8.95E+03	1.90E+03	3.60E+13
7000	9.56E+04	3.83E+04	7.51E+03	1.59E+03	3.04E+13
10000	7.56E+04	3.03E+04	5.94E+03	1.26E+03	2.41E+13
15000	5.55E+04	2.22E+04	4.36E+03	9.24E+02	1.68E+13
20000	4.47E+04	1.79E+04	3.51E+03	7.45E+02	1.24E+13
25000	3.89E+04	1.56E+04	3.05E+03	6.48E+02	9.55E+12
35000	3.37E+04	1.35E+04	2.65E+03	5.61E+02	6.68E+12
50000	3.11E+04	1.25E+04	2.45E+03	5.19E+02	5.27E+12
70000	2.97E+04	1.19E+04	2.33E+03	4.95E+02	4.88E+12
100000	2.80E+04	1.12E+04	2.20E+03	4.68E+02	4.90E+12
150000	2.55E+04	1.02E+04	2.01E+03	4.26E+02	5.09E+12
200000	2.33E+04	9.34E+03	1.83E+03	3.89E+02	5.13E+12
250000	2.12E+04	8.51E+03	1.67E+03	3.55E+02	5.07E+12
350000	1.77E+04	7.09E+03	1.39E+03	2.95E+02	4.79E+12
500000	1.34E+04	5.39E+03	1.06E+03	2.24E+02	4.28E+12
700000	9.34E+03	3.75E+03	7.35E+02	1.56E+02	3.70E+12
1000000	5.46E+03	2.19E+03	4.30E+02	9.12E+01	3.11E+12

SOURCE: CRWMS M&amp;O 1999c, disk 1 of 5, file PWR.gamma.

## 5.4 PHYSICAL PROPERTIES OF AIR-WATER VAPOR MIXTURES

The atomic density of the composition of air-water vapor mixture is required in the calculations of nitric acid production in the 21-PWR WP. The mass densities of water vapor and air are needed for computing the atomic density of the moist air composition. For this calculation, the pressure inside the failed WP is assumed to be one atmosphere (Assumption 3.11), and the relative humidity of 40%

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and 90% at temperature 30°C and 90°C are used (Assumption 3.11).

Since relative humidity is the ratio of the pressure of water vapor present to the pressure of saturated water vapor at the same temperature, the water vapor pressure in moist air is the product of the relative humidity and the saturated vapor pressure. The air pressure is then the difference of the atmospheric pressure and the water vapor pressure at the given temperature.

Given the temperature and the partial pressures of water vapor and air, the densities of water vapor and air can be determined using the ideal gas law

$$P V = n R T \quad (\text{Eq. 2})$$

where

P is the pressure of water vapor or air

V is the volume of water vapor or air

n is the number of moles of water vapor or air

R is the ideal gas constant, 8.31451E+00 joules per mole per kelvin (Parrington et al. 1996, p. 59)

T is the temperature.

Since volume equals to mass divided by density,  $V = m/\rho$ , the equation above can be rewritten for the density as

$$\rho = \frac{P m}{n R T} = \frac{P \frac{m}{n}}{R T} = \frac{P M}{R T} \quad (\text{Eq. 3})$$

where M is the molecular weight.

The density of moist air is sum of the densities of water vapor and air, and with Equation 3 the density of moist air can be expressed as

$$\begin{aligned} \rho_{ma} &= \rho_v + \rho_a \\ &= \frac{P_v M_v}{R T} + \frac{P_a M_a}{R T} \end{aligned} \quad (\text{Eq. 4})$$

where

$\rho_{ma}$  is the density of moist air

$\rho_v$  is the density of water vapor

$\rho_a$  is the density of air

$P_v$  is the partial pressure of water vapor

$P_a$  is the partial pressure of air

$M_v$  is the molecular weight of water vapor

$M_a$  is the molecular weight of air.

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Since for a given temperature the partial pressure of water vapor is the product the relative humidity and the saturated pressure, Equation 4 becomes

$$\rho_{ma} = \frac{(RH) (P_{sat}) M_v}{R T} + \frac{[P_0 - (RH) (P_{sat})] M_a}{R T} \quad (\text{Eq. 5})$$

where

RH is the relative humidity at temperature T

$P_{sat}$  is the saturated vapor pressure at temperature T

$P_0$  is the pressure at one atmosphere (Assumption 3.11).

Equation 5 provides the relationship to compute the densities of water vapor and air in a water vapor-air mixture as a function of the relative humidity and temperature at pressure  $P_0$ . From the densities and compositions of water vapor and air, atomic densities of the compositions can be calculated. The saturated vapor pressure as a function of temperature can be found in ASME 1993 (p. 54) and given in Table 12. The composition of air can be found in Weast 1979 (p. F-211) and presented in Table 13.

Table 12. Saturated Vapor Pressure Versus Temperature

Temperature ("C)	Temperature ("K)	Pressure (Pa)
10	283.15	1227
20	293.15	2336.6
30	303.15	4241.5
40	313.15	7375
50	323.15	12335.3
60	333.15	19920.2
70	343.15	31162
80	353.15	47359.8

SOURCE: ASME 1993, p. 54.

Table 13. Composition of Air

Nuclide	Volume Fraction	Atomic Mass
N <sub>2</sub>	0.78084	14.00674
O <sub>2</sub>	0.20946	15.9994
Ar	0.00934	39.948
CO <sub>2</sub>	0.00033	44.0095

SOURCE: Weast 1979, p. F-211.

The atom density of the composition of water vapor-air mixture at various relative humidity and temperature was calculated in the worksheet "AtomDensity" of the Excel file "MoistAirProperties," which is included in the compact disk as Attachment III of this document. Table 14 presents the atom density of the moist air composition at the conditions used in this calculation.

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Table 14. Atom Density (atoms/b•cm) of Composition of Water Vapor-Air Mixture

Relative Humidity and Temperature	N	Ar	C	O	H	Total
RH=40% T=30°C	3.717E-05	2.223E-07	7.854E-09	1.039E-05	8.126E-07	4.861E-05
RH=40% T=90°C	2.284E-05	1.366E-07	4.827E-09	1.172E-05	1.116E-05	4.586E-05
RH=90% T=30°C	3.638E-05	2.176E-07	7.687E-09	1.069E-05	1.828E-06	4.912E-05
RH=90% T=90°C	1.195E-05	7.148E-08	2.526E-09	1.577E-05	2.511E-05	5.290E-05

## 5.5 NITRIC ACID PRODUCTION IN THE 21-PWR WP

Radiolytic production of nitric acid in the 21-PWR WP for the time period between ten and one million years is determined from the energy deposition in the moist air filling the cavity of the WP and the G values of neutron and gamma radiation. The energy deposition rates by neutron and gamma radiation in the moist air in the lattice cell of the spent fuel rods in the central nine assemblies are calculated by MCNP. Four environment conditions are considered for the WP, all under one atmospheric pressure (Assumption 3.11). The conditions are the combinations of two relative humidities (40% and 90%) and two temperatures (30°C and 90°C). For each radiation source, there are 36 calculations each corresponding to a time step between ten and one million years.

The G values of 1.5 are used for both neutron and gamma (Assumptions 3.9 and 3.10) to determine the number of nitric acid molecules produced per second in the central nine assemblies of the WP. The results of this calculation can be scaled to compute production rate of other chemical species.

The nitric acid production rates per volume of moist air and per surface area of the fuel rods are computed from the G values and the energy depositions. Since the "f6" tally in MCNP is employed to score the energy depositions in units of MeV/g/s, the density of the moist air, the volume of the tally, and the total surface area of the fuel rods are needed. The density of the moist air for the four WP environments are presented in the worksheet "MoistAir2" of the Excel file "MoistAirProperties," which is included in the compact disk as Attachment III. The tally cell volume and the fuel rod surface area are computed below.

The volume of the unit cells outside the fuel rods in the central nine assemblies is:

$$(9)(208)(360.172)[(1.44272)^2 - \pi(0.5461)^2] \text{ cm}^3 \\ = 7.7170\text{E}+5 \text{ cm}^3.$$

The total surface area of the fuel rods is:

$$(9)(208)(360.172)(2\pi)(0.5461) \text{ cm}^2 \\ = 2.3135\text{E}+6 \text{ cm}^2.$$

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## 6. RESULTS

The section presents the results of this calculation. The outputs of this calculation are reasonable compared to the inputs, and the results are suitable for the intended use. The uncertainties are taken into account by consistently using the most conservative approach; the calculations, therefore, yield a conservatively bounding set of results.

### 6.1 DOSE RATES AND ENERGY DEPOSITIONS BY GAMMA RAYS

Radiation dose rates and energy deposition rates by gamma radiation in the unit cells outside the fuel rods in the central nine SNF assemblies of the 21-PWR WP were calculated by MCNP and presented in Tables 15 and 16. The results were calculated for the four moist air conditions at 35 time steps spanning over one million years.

Table 15. Gamma Dose Rates (rad/h) Outside PWR SNF Rods

Age (Years)	RH=40% T=90°C	RH=40% T=30°C	RH=90% T=30°C	RH=90% T=90°C
10	7.81E+04	7.82E+04	7.82E+04	7.82E+04
15	5.75E+04	5.76E+04	5.76E+04	5.76E+04
20	4.69E+04	4.69E+04	4.69E+04	4.69E+04
25	4.02E+04	4.02E+04	4.02E+04	4.02E+04
35	3.04E+04	3.04E+04	3.04E+04	3.04E+04
50	2.10E+04	2.10E+04	2.10E+04	2.10E+04
70	1.31E+04	1.31E+04	1.31E+04	1.31E+04
100	6.51E+03	6.50E+03	6.50E+03	6.50E+03
150	2.05E+03	2.05E+03	2.05E+03	2.05E+03
200	6.46E+02	6.46E+02	6.46E+02	6.46E+02
250	2.07E+02	2.07E+02	2.07E+02	2.07E+02
350	2.51E+01	2.51E+01	2.51E+01	2.51E+01
500	5.47E+00	5.47E+00	5.47E+00	5.47E+00
700	4.59E+00	4.59E+00	4.59E+00	4.59E+00
1000	4.23E+00	4.23E+00	4.23E+00	4.23E+00
1500	4.02E+00	4.02E+00	4.02E+00	4.02E+00
2000	3.78E+00	3.78E+00	3.78E+00	3.78E+00
2500	3.71E+00	3.71E+00	3.71E+00	3.71E+00
3500	3.61E+00	3.61E+00	3.61E+00	3.61E+00
5000	3.49E+00	3.49E+00	3.49E+00	3.49E+00
7000	3.36E+00	3.36E+00	3.36E+00	3.36E+00
10000	3.22E+00	3.22E+00	3.22E+00	3.22E+00
15000	3.08E+00	3.08E+00	3.08E+00	3.08E+00
20000	3.00E+00	3.00E+00	3.00E+00	3.00E+00
25000	2.95E+00	2.95E+00	2.95E+00	2.95E+00
35000	2.99E+00	2.99E+00	2.99E+00	2.99E+00
50000	3.12E+00	3.12E+00	3.12E+00	3.12E+00
70000	3.34E+00	3.34E+00	3.34E+00	3.34E+00
100000	3.64E+00	3.64E+00	3.64E+00	3.64E+00
150000	3.95E+00	3.95E+00	3.95E+00	3.95E+00
200000	3.95E+00	3.95E+00	3.95E+00	3.95E+00
250000	3.80E+00	3.80E+00	3.80E+00	3.80E+00
350000	3.30E+00	3.30E+00	3.30E+00	3.30E+00
500000	2.56E+00	2.56E+00	2.56E+00	2.56E+00
700000	1.82E+00	1.82E+00	1.82E+00	1.82E+00
1000000	1.23E+00	1.23E+00	1.23E+00	1.23E+00

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Table 16. Gamma Energy Deposition Rates (MeV/g/s) Outside PWR SNF Rods

Age (Years)	RH=40% T=90°C	RH=40% T=30°C	RH=90% T=30°C	RH=90% T=90°C
10	1.06E+09	1.04E+09	1.05E+09	1.10E+09
15	7.81E+08	7.66E+08	7.67E+08	8.09E+08
20	6.35E+08	6.23E+08	6.23E+08	6.57E+08
25	5.43E+08	5.32E+08	5.33E+08	5.62E+08
35	4.09E+08	4.01E+08	4.02E+08	4.23E+08
50	2.82E+08	2.77E+08	2.77E+08	2.92E+08
70	1.76E+08	1.72E+08	1.72E+08	1.82E+08
100	8.75E+07	8.57E+07	8.58E+07	9.05E+07
150	2.76E+07	2.70E+07	2.71E+07	2.85E+07
200	8.68E+06	8.51E+06	8.52E+06	8.98E+06
250	2.78E+06	2.73E+06	2.73E+06	2.88E+06
350	3.31E+05	3.24E+05	3.24E+05	3.42E+05
500	6.69E+04	6.56E+04	6.57E+04	6.91E+04
700	5.65E+04	5.54E+04	5.55E+04	5.83E+04
1000	5.30E+04	5.20E+04	5.20E+04	5.48E+04
1500	5.15E+04	5.04E+04	5.05E+04	5.32E+04
2000	4.90E+04	4.80E+04	4.81E+04	5.06E+04
2500	4.84E+04	4.75E+04	4.75E+04	5.01E+04
3500	4.74E+04	4.64E+04	4.65E+04	4.90E+04
5000	4.60E+04	4.51E+04	4.52E+04	4.76E+04
7000	4.45E+04	4.36E+04	4.36E+04	4.60E+04
10000	4.30E+04	4.22E+04	4.22E+04	4.45E+04
15000	4.16E+04	4.07E+04	4.08E+04	4.30E+04
20000	4.08E+04	4.00E+04	4.00E+04	4.22E+04
25000	4.04E+04	3.96E+04	3.96E+04	4.18E+04
35000	4.11E+04	4.03E+04	4.04E+04	4.25E+04
50000	4.33E+04	4.24E+04	4.25E+04	4.48E+04
70000	4.66E+04	4.57E+04	4.57E+04	4.82E+04
100000	5.10E+04	5.00E+04	5.01E+04	5.28E+04
150000	5.56E+04	5.44E+04	5.45E+04	5.74E+04
200000	5.56E+04	5.45E+04	5.45E+04	5.75E+04
250000	5.35E+04	5.24E+04	5.25E+04	5.53E+04
350000	4.66E+04	4.56E+04	4.57E+04	4.81E+04
500000	3.60E+04	3.53E+04	3.54E+04	3.73E+04
700000	2.55E+04	2.50E+04	2.50E+04	2.64E+04
1000000	1.72E+04	1.68E+04	1.68E+04	1.77E+04

## 6.2 DOSE RATES AND ENERGY DEPOSITIONS BY NEUTRONS

Similar to the gamma results, dose rates and energy deposition rates by neutron radiation in the unit cells outside the fuel rods in the central nine SNF assemblies of the 21-PWR WP were calculated by MCNP and presented in Tables 17 and 18. The results were calculated for the four moist air conditions at 35 time steps spanning over one million years.

Table 17. Neutron Dose Rates (rad/h) Outside PWR SNF Rods

Age (Years)	n/s/ass	n/s/WP	RH=40% T=90°C	RH=40% T=30°C	RH=90% T=30°C	RH=90% T=90°C
10	3.4640E+08	7.2744E+09	3.5059E+00	3.5010E+00	3.5014E+00	3.5089E+00
15	2.8770E+08	6.0417E+09	2.9118E+00	2.9078E+00	2.9081E+00	2.9143E+00
20	2.3931E+08	5.0255E+09	2.4220E+00	2.4187E+00	2.4189E+00	2.4241E+00

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25	1.9933E+08	4.1859E+09	2.0174E+00	2.0146E+00	2.0148E+00	2.0191E+00
35	1.3899E+08	2.9188E+09	1.4067E+00	1.4048E+00	1.4049E+00	1.4079E+00
50	8.2570E+07	1.7340E+09	8.3569E-01	8.3453E-01	8.3462E-01	8.3640E-01
70	4.3246E+07	9.0817E+08	4.3769E-01	4.3708E-01	4.3713E-01	4.3806E-01
100	1.9766E+07	4.1509E+08	2.0005E-01	1.9977E-01	1.9979E-01	2.0022E-01
150	1.0018E+07	2.1038E+08	1.0139E-01	1.0125E-01	1.0126E-01	1.0148E-01
200	8.1960E+06	1.7212E+08	8.2951E-02	8.2836E-02	8.2845E-02	8.3022E-02
250	7.6199E+06	1.6002E+08	7.7121E-02	7.7014E-02	7.7022E-02	7.7187E-02
350	7.0603E+06	1.4827E+08	7.1457E-02	7.1358E-02	7.1365E-02	7.1518E-02
500	6.5264E+06	1.3705E+08	6.6054E-02	6.5962E-02	6.5969E-02	6.6110E-02
700	6.0621E+06	1.2730E+08	6.1354E-02	6.1269E-02	6.1276E-02	6.1407E-02
1000	5.5610E+06	1.1678E+08	5.6283E-02	5.6204E-02	5.6210E-02	5.6331E-02
1500	5.0051E+06	1.0511E+08	5.0656E-02	5.0586E-02	5.0591E-02	5.0700E-02
2000	4.6326E+06	9.7285E+07	4.6886E-02	4.6821E-02	4.6826E-02	4.6926E-02
2500	4.3403E+06	9.1146E+07	4.3928E-02	4.3867E-02	4.3872E-02	4.3966E-02
3500	3.8816E+06	8.1514E+07	3.9286E-02	3.9231E-02	3.9235E-02	3.9319E-02
5000	3.3339E+06	7.0012E+07	3.3742E-02	3.3695E-02	3.3699E-02	3.3771E-02
7000	2.7627E+06	5.8017E+07	2.7961E-02	2.7922E-02	2.7925E-02	2.7985E-02
10000	2.1410E+06	4.4961E+07	2.1669E-02	2.1639E-02	2.1641E-02	2.1687E-02
15000	1.5146E+06	3.1807E+07	1.5329E-02	1.5308E-02	1.5310E-02	1.5342E-02
20000	1.1779E+06	2.4736E+07	1.1921E-02	1.1905E-02	1.1906E-02	1.1932E-02
25000	9.9340E+05	2.0861E+07	1.0054E-02	1.0040E-02	1.0041E-02	1.0063E-02
35000	8.2520E+05	1.7329E+07	8.3518E-03	8.3402E-03	8.3411E-03	8.3589E-03
50000	7.4130E+05	1.5567E+07	7.5027E-03	7.4922E-03	7.4930E-03	7.5091E-03
70000	6.9530E+05	1.4601E+07	7.0371E-03	7.0273E-03	7.0281E-03	7.0431E-03
100000	6.4920E+05	1.3633E+07	6.5705E-03	6.5614E-03	6.5621E-03	6.5761E-03
150000	5.8910E+05	1.2371E+07	5.9623E-03	5.9540E-03	5.9546E-03	5.9673E-03
200000	5.3750E+05	1.1288E+07	5.4400E-03	5.4325E-03	5.4330E-03	5.4447E-03
250000	4.9203E+05	1.0333E+07	4.9798E-03	4.9729E-03	4.9734E-03	4.9841E-03
350000	4.1051E+05	8.6207E+06	4.1548E-03	4.1490E-03	4.1494E-03	4.1583E-03
500000	3.1361E+05	6.5858E+06	3.1740E-03	3.1696E-03	3.1700E-03	3.1767E-03
700000	2.2001E+05	4.6202E+06	2.2267E-03	2.2236E-03	2.2239E-03	2.2286E-03
1000000	1.3088E+05	2.7485E+06	1.3246E-03	1.3228E-03	1.3229E-03	1.3258E-03

Table 18. Neutron Energy Deposition Rates (MeV/g/s ) Outside PWR SNF Rods

Age (Years)	n/s/ass	n/s/WP	RH=40% T=90°C	RH=40% T=30°C	RH=90% T=30°C	RH=90% T=90°C
10	3.4640E+08	7.2744E+09	1.2995E+04	5.0720E+03	5.6575E+03	2.6684E+04
15	2.8770E+08	6.0417E+09	1.0793E+04	4.2125E+03	4.6988E+03	2.2162E+04
20	2.3931E+08	5.0255E+09	8.9774E+03	3.5040E+03	3.9085E+03	1.8435E+04
25	1.9933E+08	4.1859E+09	7.4776E+03	2.9186E+03	3.2555E+03	1.5355E+04
35	1.3899E+08	2.9188E+09	5.2140E+03	2.0351E+03	2.2700E+03	1.0707E+04
50	8.2570E+07	1.7340E+09	3.0975E+03	1.2090E+03	1.3486E+03	6.3606E+03
70	4.3246E+07	9.0817E+08	1.6223E+03	6.3321E+02	7.0631E+02	3.3313E+03
100	1.9766E+07	4.1509E+08	7.4149E+02	2.8942E+02	3.2282E+02	1.5226E+03
150	1.0018E+07	2.1038E+08	3.7581E+02	1.4668E+02	1.6362E+02	7.7171E+02
200	8.1960E+06	1.7212E+08	3.0746E+02	1.2001E+02	1.3386E+02	6.3136E+02
250	7.6199E+06	1.6002E+08	2.8585E+02	1.1157E+02	1.2445E+02	5.8698E+02
350	7.0603E+06	1.4827E+08	2.6486E+02	1.0338E+02	1.1531E+02	5.4387E+02
500	6.5264E+06	1.3705E+08	2.4483E+02	9.5560E+01	1.0659E+02	5.0274E+02
700	6.0621E+06	1.2730E+08	2.2741E+02	8.8762E+01	9.9008E+01	4.6698E+02
1000	5.5610E+06	1.1678E+08	2.0861E+02	8.1425E+01	9.0824E+01	4.2838E+02
1500	5.0051E+06	1.0511E+08	1.8776E+02	7.3285E+01	8.1745E+01	3.8555E+02
2000	4.6326E+06	9.7285E+07	1.7379E+02	6.7831E+01	7.5661E+01	3.5686E+02
2500	4.3403E+06	9.1146E+07	1.6282E+02	6.3551E+01	7.0887E+01	3.3434E+02
3500	3.8816E+06	8.1514E+07	1.4561E+02	5.6835E+01	6.3395E+01	2.9901E+02
5000	3.3339E+06	7.0012E+07	1.2507E+02	4.8815E+01	5.4450E+01	2.5682E+02

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7000	2.7627E+06	5.8017E+07	1.0364E+02	4.0452E+01	4.5121E+01	2.1282E+02
10000	2.1410E+06	4.4961E+07	8.0317E+01	3.1349E+01	3.4967E+01	1.6493E+02
15000	1.5146E+06	3.1807E+07	5.6818E+01	2.2177E+01	2.4737E+01	1.1667E+02
20000	1.1779E+06	2.4736E+07	4.4187E+01	1.7247E+01	1.9238E+01	9.0736E+01
25000	9.9340E+05	2.0861E+07	3.7266E+01	1.4545E+01	1.6224E+01	7.6524E+01
35000	8.2520E+05	1.7329E+07	3.0956E+01	1.2083E+01	1.3477E+01	6.3567E+01
50000	7.4130E+05	1.5567E+07	2.7809E+01	1.0854E+01	1.2107E+01	5.7104E+01
70000	6.9530E+05	1.4601E+07	2.6083E+01	1.0181E+01	1.1356E+01	5.3561E+01
100000	6.4920E+05	1.3633E+07	2.4354E+01	9.5057E+00	1.0603E+01	5.0009E+01
150000	5.8910E+05	1.2371E+07	2.2099E+01	8.6257E+00	9.6213E+00	4.5380E+01
200000	5.3750E+05	1.1288E+07	2.0164E+01	7.8701E+00	8.7786E+00	4.1405E+01
250000	4.9203E+05	1.0333E+07	1.8458E+01	7.2044E+00	8.0360E+00	3.7902E+01
350000	4.1051E+05	8.6207E+06	1.5400E+01	6.0107E+00	6.7046E+00	3.1623E+01
500000	3.1361E+05	6.5858E+06	1.1765E+01	4.5919E+00	5.1220E+00	2.4158E+01
700000	2.2001E+05	4.6202E+06	8.2534E+00	3.2214E+00	3.5933E+00	1.6948E+01
1000000	1.3088E+05	2.7485E+06	4.9098E+00	1.9164E+00	2.1376E+00	1.0082E+01

### 6.3 NITRIC ACID PRODUCTION IN THE 21-PWR WP

The gamma and neutron energy deposition rates in the unit cells presented in the previous section are used to determine the radiolytic production rates of nitric acid for the time period out to one million years. With the G values of 1.5 for both neutron and gamma radiation, production rates per moist air volume and per fuel rods surface area were determined. Tables 19 and 20 present the nitric acid production rates per volume at four moist air conditions for gamma rays and neutrons, respectively. Table 21 summarizes the maximum total production rates per moist air volume, which occur at the relative humidity of 40% and temperature of 30°C. Similarly, Table 22 gives the total production rates per surface area of the fuel rods. Results in Table 22 are graphically depicted in Figure 5. It is obvious that the total nitric production is dominated by gamma radiation and the production by neutron is negligible.

The results for nitric acid production can be extended to determine the production of other chemical species so long as G values are known. As a matter of fact, the yield of any particular chemical species is equal to its G value divided by 1.5 and multiplied by the yield.

Table 19. Nitric Acid Production Rate per Moist Air Volume (molecules/cm<sup>3</sup>) by Gamma Rays

Years	RH=40% T=90°C	RH=40% T=30°C	RH=90% T=30°C	RH=90% T=90°C
10	4.3819E+17	5.7144E+17	5.6721E+17	3.8751E+17
15	3.2157E+17	4.1943E+17	4.1635E+17	2.8441E+17
20	2.6147E+17	3.4076E+17	3.3826E+17	2.3108E+17
25	2.2350E+17	2.9129E+17	2.8915E+17	1.9753E+17
35	1.6846E+17	2.1950E+17	2.1789E+17	1.4885E+17
50	1.1619E+17	1.5135E+17	1.5023E+17	1.0263E+17
70	7.2333E+16	9.4230E+16	9.3540E+16	6.3898E+16
100	3.6028E+16	4.6913E+16	4.6569E+16	3.1812E+16
150	1.1356E+16	1.4793E+16	1.4685E+16	1.0031E+16
200	3.5734E+15	4.6551E+15	4.6210E+15	3.1567E+15
250	1.1453E+15	1.4918E+15	1.4808E+15	1.0116E+15
350	1.3606E+14	1.7727E+14	1.7596E+14	1.2015E+14
500	2.7550E+13	3.5914E+13	3.5651E+13	2.4301E+13
700	2.3251E+13	3.0310E+13	3.0088E+13	2.0510E+13
1000	2.1822E+13	2.8440E+13	2.8230E+13	1.9258E+13

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1500	2.1186E+13	2.7605E+13	2.7402E+13	1.8704E+13
2000	2.0172E+13	2.6281E+13	2.6088E+13	1.7812E+13
2500	1.9943E+13	2.5981E+13	2.5789E+13	1.7611E+13
3500	1.9499E+13	2.5401E+13	2.5214E+13	1.7219E+13
5000	1.8952E+13	2.4690E+13	2.4508E+13	1.6737E+13
7000	1.8310E+13	2.3851E+13	2.3676E+13	1.6171E+13
10000	1.7709E+13	2.3069E+13	2.2899E+13	1.5640E+13
15000	1.7106E+13	2.2284E+13	2.2120E+13	1.5108E+13
20000	1.6792E+13	2.1874E+13	2.1713E+13	1.4832E+13
25000	1.6626E+13	2.1659E+13	2.1500E+13	1.4685E+13
35000	1.6934E+13	2.2060E+13	2.1898E+13	1.4958E+13
50000	1.7832E+13	2.3227E+13	2.3057E+13	1.5751E+13
70000	1.9189E+13	2.4992E+13	2.4808E+13	1.6949E+13
100000	2.1008E+13	2.7362E+13	2.7161E+13	1.8556E+13
150000	2.2875E+13	2.9793E+13	2.9574E+13	2.0204E+13
200000	2.2886E+13	2.9807E+13	2.9589E+13	2.0214E+13
250000	2.2033E+13	2.8696E+13	2.8486E+13	1.9460E+13
350000	1.9166E+13	2.4963E+13	2.4780E+13	1.6928E+13
500000	1.4837E+13	1.9325E+13	1.9183E+13	1.3104E+13
700000	1.0507E+13	1.3686E+13	1.3586E+13	9.2808E+12
1000000	7.0654E+12	9.2030E+12	9.1354E+12	6.2403E+12

Table 20. Nitric Acid Production Rate per Moist Air Volume (molecules/lycm<sup>3</sup>) by Neutrons

Years	RH=40% T=90°C	RH=40% T=30°C	RH=90% T=30°C	RH=90% T=90°C
10	5.3491E+12	2.7760E+12	3.0696E+12	9.3849E+12
15	4.4427E+12	2.3056E+12	2.5495E+12	7.7946E+12
20	3.6954E+12	1.9178E+12	2.1206E+12	6.4835E+12
25	3.0781E+12	1.5974E+12	1.7664E+12	5.4004E+12
35	2.1463E+12	1.1138E+12	1.2317E+12	3.7656E+12
50	1.2751E+12	6.6169E+11	7.3170E+11	2.2370E+12
70	6.6781E+11	3.4656E+11	3.8322E+11	1.1716E+12
100	3.0523E+11	1.5840E+11	1.7516E+11	5.3551E+11
150	1.5470E+11	8.0282E+10	8.8775E+10	2.7141E+11
200	1.2656E+11	6.5681E+10	7.2629E+10	2.2205E+11
250	1.1767E+11	6.1064E+10	6.7524E+10	2.0644E+11
350	1.0903E+11	5.6579E+10	6.2565E+10	1.9128E+11
500	1.0078E+11	5.2301E+10	5.7834E+10	1.7682E+11
700	9.3611E+10	4.8580E+10	5.3719E+10	1.6424E+11
1000	8.5873E+10	4.4564E+10	4.9279E+10	1.5066E+11
1500	7.7289E+10	4.0110E+10	4.4353E+10	1.3560E+11
2000	7.1537E+10	3.7124E+10	4.1052E+10	1.2551E+11
2500	6.7023E+10	3.4782E+10	3.8462E+10	1.1759E+11
3500	5.9940E+10	3.1106E+10	3.4397E+10	1.0516E+11
5000	5.1482E+10	2.6717E+10	2.9543E+10	9.0324E+10
7000	4.2662E+10	2.2140E+10	2.4482E+10	7.4849E+10
10000	3.3061E+10	1.7157E+10	1.8972E+10	5.8005E+10
15000	2.3389E+10	1.2138E+10	1.3422E+10	4.1035E+10
20000	1.8189E+10	9.4394E+09	1.0438E+10	3.1912E+10
25000	1.5340E+10	7.9608E+09	8.8030E+09	2.6914E+10
35000	1.2743E+10	6.6129E+09	7.3125E+09	2.2357E+10
50000	1.1447E+10	5.9406E+09	6.5690E+09	2.0084E+10
70000	1.0737E+10	5.5720E+09	6.1614E+09	1.8838E+10
100000	1.0025E+10	5.2025E+09	5.7529E+09	1.7589E+10
150000	9.0969E+09	4.7209E+09	5.2203E+09	1.5960E+10
200000	8.3001E+09	4.3074E+09	4.7631E+09	1.4562E+10
250000	7.5980E+09	3.9430E+09	4.3601E+09	1.3330E+10
350000	6.3391E+09	3.2897E+09	3.6377E+09	1.1122E+10
500000	4.8428E+09	2.5132E+09	2.7791E+09	8.4965E+09
700000	3.3974E+09	1.7631E+09	1.9496E+09	5.9607E+09
1000000	2.0211E+09	1.0488E+09	1.1598E+09	3.5459E+09

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Table 21. Nitric Acid Production Rate per Volume (molecules/y/cm<sup>3</sup>) at 40% RH and 30°C

Years	Gammas	Neutrons	Total
10	5.7144E+17	2.7760E+12	5.7145E+17
15	4.1943E+17	2.3056E+12	4.1943E+17
20	3.4076E+17	1.9178E+12	3.4076E+17
25	2.9129E+17	1.5974E+12	2.9129E+17
35	2.1950E+17	1.1138E+12	2.1950E+17
50	1.5135E+17	6.6169E+11	1.5135E+17
70	9.4230E+16	3.4656E+11	9.4230E+16
100	4.6913E+16	1.5840E+11	4.6913E+16
150	1.4793E+16	8.0282E+10	1.4793E+16
200	4.6551E+15	6.5681E+10	4.6552E+15
250	1.4918E+15	6.1064E+10	1.4919E+15
350	1.7727E+14	5.6579E+10	1.7732E+14
500	3.5914E+13	5.2301E+10	3.5967E+13
700	3.0310E+13	4.8580E+10	3.0358E+13
1000	2.8440E+13	4.4564E+10	2.8484E+13
1500	2.7605E+13	4.0110E+10	2.7645E+13
2000	2.6281E+13	3.7124E+10	2.6318E+13
2500	2.5981E+13	3.4782E+10	2.6015E+13
3500	2.5401E+13	3.1106E+10	2.5432E+13
5000	2.4690E+13	2.6717E+10	2.4716E+13
7000	2.3851E+13	2.2140E+10	2.3873E+13
10000	2.3069E+13	1.7157E+10	2.3086E+13
15000	2.2284E+13	1.2138E+10	2.2296E+13
20000	2.1874E+13	9.4394E+09	2.1883E+13
25000	2.1659E+13	7.9608E+09	2.1667E+13
35000	2.2060E+13	6.6129E+09	2.2066E+13
50000	2.3227E+13	5.9406E+09	2.3233E+13
70000	2.4992E+13	5.5720E+09	2.4997E+13
100000	2.7362E+13	5.2025E+09	2.7367E+13
150000	2.9793E+13	4.7209E+09	2.9797E+13
200000	2.9807E+13	4.3074E+09	2.9812E+13
250000	2.8696E+13	3.9430E+09	2.8700E+13
350000	2.4963E+13	3.2897E+09	2.4967E+13
500000	1.9325E+13	2.5132E+09	1.9327E+13
700000	1.3686E+13	1.7631E+09	1.3688E+13
1000000	9.2030E+12	1.0488E+09	9.2040E+12

Table 22. Nitric Acid Production Rate per Fuel Rod Area (molecules/y/cm<sup>2</sup>) at 40% RH and 30°C

Years	Gammas	Neutrons	Total
10	1.9061E+17	9.2596E+11	1.9061E+17
15	1.3991E+17	7.6905E+11	1.3991E+17
20	1.1366E+17	6.3970E+11	1.1367E+17
25	9.7163E+16	5.3283E+11	9.7164E+16
35	7.3218E+16	3.7153E+11	7.3219E+16
50	5.0484E+16	2.2072E+11	5.0484E+16
70	3.1432E+16	1.1560E+11	3.1432E+16
100	1.5648E+16	5.2836E+10	1.5649E+16
150	4.9345E+15	2.6779E+10	4.9345E+15
200	1.5528E+15	2.1909E+10	1.5528E+15
250	4.9761E+14	2.0369E+10	4.9763E+14
350	5.9130E+13	1.8873E+10	5.9149E+13
500	1.1980E+13	1.7446E+10	1.1997E+13
700	1.0110E+13	1.6205E+10	1.0126E+13

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1000	9.4865E+12	1.4865E+10	9.5013E+12
1500	9.2081E+12	1.3379E+10	9.2214E+12
2000	8.7664E+12	1.2383E+10	8.7788E+12
2500	8.6662E+12	1.1602E+10	8.6778E+12
3500	8.4729E+12	1.0376E+10	8.4833E+12
5000	8.2356E+12	8.9118E+09	8.2445E+12
7000	7.9559E+12	7.3850E+09	7.9633E+12
10000	7.6950E+12	5.7231E+09	7.7007E+12
15000	7.4332E+12	4.0487E+09	7.4372E+12
20000	7.2963E+12	3.1486E+09	7.2994E+12
25000	7.2245E+12	2.6555E+09	7.2272E+12
35000	7.3583E+12	2.2058E+09	7.3605E+12
50000	7.7477E+12	1.9816E+09	7.7497E+12
70000	8.3363E+12	1.8586E+09	8.3382E+12
100000	9.1268E+12	1.7354E+09	9.1286E+12
150000	9.9378E+12	1.5747E+09	9.9393E+12
200000	9.9427E+12	1.4368E+09	9.9441E+12
250000	9.5721E+12	1.3152E+09	9.5734E+12
350000	8.3268E+12	1.0973E+09	8.3279E+12
500000	6.4461E+12	8.3831E+08	6.4469E+12
700000	4.5651E+12	5.8811E+08	4.5657E+12
1000000	3.0698E+12	3.4985E+08	3.0701E+12

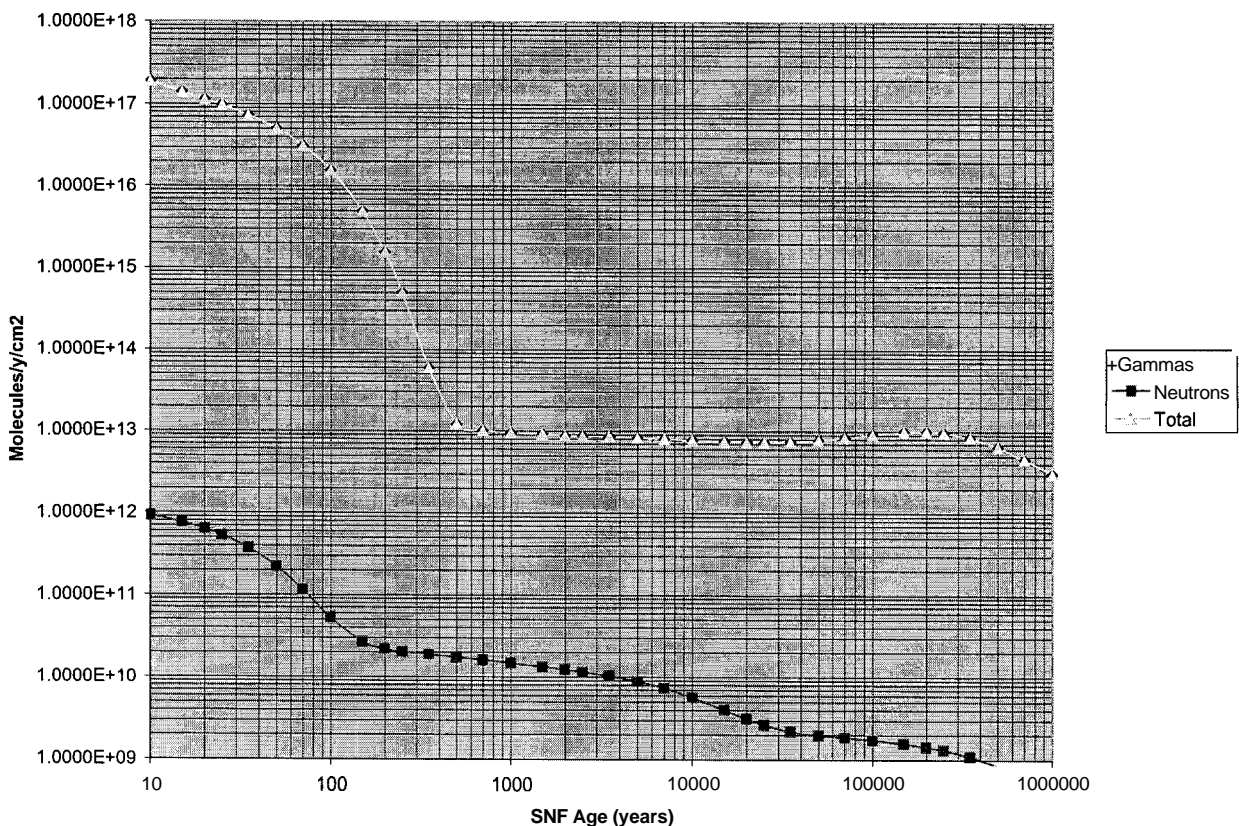


Figure 5. Nitric Acid Production Rate per Fuel Rod Area

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#### 6.4 THERMAL POWER OF THE AVERAGE PWR SNF ASSEMBLY

The thermal power (decay heat rate) of the average PWR SNF assembly for the time period between one and one million years are obtained from the file Waste.Stream.E5.R1.B9.cut in disk 3 of 5 (CRWMS M&O 1999c), and presented in Table 23. Figure 6 graphically displays the data in the table.

The thermal power is comprised of three components, the light elements, the actinides, and the fission products. The light elements come from the incore spacers and grid supports; the heat of the light elements is basically due to gamma and beta decays of the radioactive elements. The biggest contributor is the  $\text{Co}^{60}$  isotope, which has a half-life of 5.27 years (Parrington et al. 1996, p. 25). The actinides are produced transmutation and decay of the heavy isotopes. The heat of the actinides comes from alpha, beta, and gamma decays and even neutrons from spontaneous fission. The fission products are the results of the nuclear fission process. The heat of the fission products is from gamma and beta decays. The major contributors to decay heat from this component are the  $\text{Sr}^{90}$ - $\text{Y}^{90}$  and the  $\text{Cs}^{137}$ - $\text{Ba}^{137}$  pairs. The first pair emits betas and the second pair emits gammas.  $\text{Sr}^{90}$  has a half-life of 28.78 years and  $\text{Cs}^{137}$  has a half-life of 30.07 years (Parrington et al. 1996, pp. 29 and 34).

It is evident from Figure 6 that for the first 60 years the thermal power from the fission products is the dominant component. After this time, decay heat from the actinides is orders of magnitude larger than the other two components.

Table 23. Thermal Power (watts) of the Average PWR SNF Assembly to One Million Years

Age (years)	Light Elements	Actinides	Fission Products	Total/Assembly	Total/WP
1	1.0000E+02	5.1600E+02	5.0400E+03	5.6560E+03	1.1878E+05
2	8.1100E+01	2.9600E+02	2.8000E+03	3.1771E+03	6.6719E+04
3	7.0500E+01	2.4900E+02	1.8100E+03	2.1295E+03	4.4720E+04
4	6.1600E+01	2.3800E+02	1.3000E+03	1.5996E+03	3.3592E+04
5	5.3900E+01	2.3400E+02	1.0300E+03	1.3179E+03	2.7676E+04
6	4.7100E+01	2.3300E+02	8.6600E+02	1.1461E+03	2.4068E+04
7	4.1300E+01	2.3100E+02	7.6700E+02	1.0393E+03	2.1825E+04
8	3.6100E+01	2.3000E+02	7.0000E+02	9.6610E+02	2.0288E+04
9	3.1700E+01	2.2800E+02	6.5300E+02	9.1270E+02	1.9167E+04
10	2.7700E+01	2.2700E+02	6.1700E+02	8.7170E+02	1.8306E+04
11	2.4300E+01	2.2600E+02	5.8800E+02	8.3830E+02	1.7604E+04
12	2.1300E+01	2.2400E+02	5.6400E+02	8.0930E+02	1.6995E+04
13	1.8700E+01	2.2300E+02	5.4300E+02	7.8470E+02	1.6479E+04
14	1.6400E+01	2.2200E+02	5.2500E+02	7.6340E+02	1.6031E+04
15	1.4300E+01	2.2000E+02	5.0800E+02	7.4230E+02	1.5588E+04
16	1.2600E+01	2.1900E+02	4.9300E+02	7.2460E+02	1.5217E+04
17	1.1000E+01	2.1800E+02	4.7900E+02	7.0800E+02	1.4868E+04
18	9.6600E+00	2.1600E+02	4.6600E+02	6.9166E+02	1.4525E+04
19	8.4700E+00	2.1500E+02	4.5400E+02	6.7747E+02	1.4227E+04
20	7.4300E+00	2.1400E+02	4.4200E+02	6.6343E+02	1.3932E+04
21	6.5200E+00	2.1200E+02	4.3000E+02	6.4852E+02	1.3619E+04
22	5.7200E+00	2.1100E+02	4.1900E+02	6.3572E+02	1.3350E+04
23	5.0100E+00	2.1000E+02	4.0900E+02	6.2401E+02	1.3104E+04
24	4.4000E+00	2.0900E+02	3.9900E+02	6.1240E+02	1.2860E+04
25	3.8600E+00	2.0700E+02	3.8900E+02	5.9986E+02	1.2597E+04
26	3.3900E+00	2.0600E+02	3.7900E+02	5.8839E+02	1.2356E+04

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27	2.9700E+00	2.0500E+02	3.7000E+02	5.7797E+02	1.2137E+04
28	2.6100E+00	2.0400E+02	3.6100E+02	5.6761E+02	1.1920E+04
29	2.2900E+00	2.0200E+02	3.5200E+02	5.5629E+02	1.1682E+04
30	2.0100E+00	2.0100E+02	3.4300E+02	5.4601E+02	1.1466E+04
31	1.7700E+00	2.0000E+02	3.3500E+02	5.3677E+02	1.1272E+04
32	1.5500E+00	1.9900E+02	3.2700E+02	5.2755E+02	1.1079E+04
33	1.3600E+00	1.9800E+02	3.1900E+02	5.1836E+02	1.0886E+04
34	1.2000E+00	1.9700E+02	3.1100E+02	5.0920E+02	1.0693E+04
35	1.0500E+00	1.9500E+02	3.0300E+02	4.9905E+02	1.0480E+04
36	9.2800E-01	1.9400E+02	2.9600E+02	4.9093E+02	1.0309E+04
37	8.1600E-01	1.9300E+02	2.8900E+02	4.8282E+02	1.0139E+04
38	7.1900E-01	1.9200E+02	2.8200E+02	4.7472E+02	9.9691E+03
39	6.3300E-01	1.9100E+02	2.7500E+02	4.6663E+02	9.7993E+03
40	5.5800E-01	1.9000E+02	2.6900E+02	4.5956E+02	9.6507E+03
41	4.9300E-01	1.8900E+02	2.6200E+02	4.5149E+02	9.4814E+03
42	4.3500E-01	1.8800E+02	2.5600E+02	4.4444E+02	9.3331E+03
43	3.8400E-01	1.8700E+02	2.5000E+02	4.3738E+02	9.1851E+03
44	3.4000E-01	1.8600E+02	2.4400E+02	4.3034E+02	9.0371E+03
45	3.0100E-01	1.8500E+02	2.3800E+02	4.2330E+02	8.8893E+03
46	2.6700E-01	1.8400E+02	2.3200E+02	4.1627E+02	8.7416E+03
47	2.3700E-01	1.8300E+02	2.2700E+02	4.1024E+02	8.6150E+03
48	2.1100E-01	1.8200E+02	2.2100E+02	4.0321E+02	8.4674E+03
49	1.8800E-01	1.8100E+02	2.1600E+02	3.9719E+02	8.3409E+03
50	1.6700E-01	1.8000E+02	2.1100E+02	3.9117E+02	8.2145E+03
51	1.5000E-01	1.7900E+02	2.0600E+02	3.8515E+02	8.0882E+03
52	1.3400E-01	1.7800E+02	2.0100E+02	3.7913E+02	7.9618E+03
53	1.2000E-01	1.7700E+02	1.9600E+02	3.7312E+02	7.8355E+03
54	1.0800E-01	1.7600E+02	1.9100E+02	3.6711E+02	7.7093E+03
55	9.7700E-02	1.7500E+02	1.8700E+02	3.6210E+02	7.6041E+03
56	8.8400E-02	1.7400E+02	1.8300E+02	3.5709E+02	7.4989E+03
57	8.0300E-02	1.7300E+02	1.7800E+02	3.5108E+02	7.3727E+03
58	7.3100E-02	1.7300E+02	1.7400E+02	3.4707E+02	7.2885E+03
59	6.6800E-02	1.7200E+02	1.7000E+02	3.4207E+02	7.1834E+03
60	6.1300E-02	1.7100E+02	1.6600E+02	3.3706E+02	7.0783E+03
61	5.6400E-02	1.7000E+02	1.6200E+02	3.3206E+02	6.9732E+03
62	5.2100E-02	1.6900E+02	1.5800E+02	3.2705E+02	6.8681E+03
63	4.8400E-02	1.6800E+02	1.5400E+02	3.2205E+02	6.7630E+03
64	4.5100E-02	1.6800E+02	1.5100E+02	3.1905E+02	6.6999E+03
65	4.2100E-02	1.6700E+02	1.4700E+02	3.1404E+02	6.5949E+03
66	3.9600E-02	1.6600E+02	1.4400E+02	3.1004E+02	6.5108E+03
67	3.7300E-02	1.6500E+02	1.4000E+02	3.0504E+02	6.4058E+03
68	3.5300E-02	1.6400E+02	1.3700E+02	3.0104E+02	6.3217E+03
69	3.3500E-02	1.6400E+02	1.3400E+02	2.9803E+02	6.2587E+03
70	3.2000E-02	1.6300E+02	1.3000E+02	2.9303E+02	6.1537E+03
71	3.0600E-02	1.6200E+02	1.2700E+02	2.8903E+02	6.0696E+03
72	2.9300E-02	1.6200E+02	1.2400E+02	2.8603E+02	6.0066E+03
73	2.8300E-02	1.6100E+02	1.2100E+02	2.8203E+02	5.9226E+03
74	2.7300E-02	1.6000E+02	1.1900E+02	2.7903E+02	5.8596E+03
75	2.6400E-02	1.5900E+02	1.1600E+02	2.7503E+02	5.7756E+03
76	2.5700E-02	1.5900E+02	1.1300E+02	2.7203E+02	5.7125E+03
77	2.5000E-02	1.5800E+02	1.1000E+02	2.6803E+02	5.6285E+03
78	2.4400E-02	1.5700E+02	1.0800E+02	2.6502E+02	5.5655E+03
79	2.3800E-02	1.5700E+02	1.0500E+02	2.6202E+02	5.5025E+03
80	2.3300E-02	1.5600E+02	1.0300E+02	2.5902E+02	5.4395E+03
81	2.2900E-02	1.5500E+02	1.0000E+02	2.5502E+02	5.3555E+03
82	2.2500E-02	1.5500E+02	9.8000E+01	2.5302E+02	5.3135E+03
83	2.2100E-02	1.5400E+02	9.5700E+01	2.4972E+02	5.2442E+03
84	2.1800E-02	1.5300E+02	9.3400E+01	2.4642E+02	5.1749E+03

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85	2.1500E-02	1.5300E+02	9.1200E+01	2.4422E+02	5.1287E+03
86	2.1200E-02	1.5200E+02	8.9100E+01	2.4112E+02	5.0635E+03
87	2.0900E-02	1.5200E+02	8.7000E+01	2.3902E+02	5.0194E+03
88	2.0700E-02	1.5100E+02	8.4900E+01	2.3592E+02	4.9543E+03
89	2.0500E-02	1.5000E+02	8.2900E+01	2.3292E+02	4.8913E+03
90	2.0300E-02	1.5000E+02	8.1000E+01	2.3102E+02	4.8514E+03
91	2.0100E-02	1.4900E+02	7.9100E+01	2.2812E+02	4.7905E+03
92	1.9900E-02	1.4900E+02	7.7200E+01	2.2622E+02	4.7506E+03
93	1.9800E-02	1.4800E+02	7.5400E+01	2.2342E+02	4.6918E+03
94	1.9600E-02	1.4700E+02	7.3600E+01	2.2062E+02	4.6330E+03
95	1.9500E-02	1.4700E+02	7.1900E+01	2.1892E+02	4.5973E+03
96	1.9400E-02	1.4600E+02	7.0200E+01	2.1622E+02	4.5406E+03
97	1.9200E-02	1.4600E+02	6.8500E+01	2.1452E+02	4.5049E+03
98	1.9100E-02	1.4500E+02	6.6900E+01	2.1192E+02	4.4503E+03
99	1.9000E-02	1.4500E+02	6.5400E+01	2.1042E+02	4.4188E+03
100	1.8900E-02	1.4400E+02	6.3800E+01	2.0782E+02	4.3642E+03
110	1.7900E-02	1.3900E+02	5.0300E+01	1.8932E+02	3.9757E+03
120	1.7100E-02	1.3400E+02	3.9700E+01	1.7372E+02	3.6481E+03
130	1.6500E-02	1.3000E+02	3.1300E+01	1.6132E+02	3.3876E+03
140	1.5800E-02	1.2600E+02	2.4700E+01	1.5072E+02	3.1650E+03
150	1.5200E-02	1.2200E+02	1.9400E+01	1.4142E+02	2.9697E+03
160	1.4700E-02	1.1900E+02	1.5300E+01	1.3431E+02	2.8206E+03
170	1.4200E-02	1.1500E+02	1.2100E+01	1.2711E+02	2.6694E+03
180	1.3700E-02	1.1200E+02	9.5400E+00	1.2155E+02	2.5526E+03
190	1.3300E-02	1.0900E+02	7.5300E+00	1.1654E+02	2.4474E+03
200	1.2900E-02	1.0700E+02	5.9400E+00	1.1295E+02	2.3720E+03
250	1.1200E-02	9.4900E+01	1.8200E+00	9.6731E+01	2.0314E+03
300	1.0100E-02	8.5600E+01	5.6800E-01	8.6178E+01	1.8097E+03
350	9.2200E-03	7.8000E+01	1.8300E-01	7.8192E+01	1.6420E+03
400	8.6300E-03	7.1700E+01	6.4800E-02	7.1773E+01	1.5072E+03
450	8.2100E-03	6.6400E+01	2.8400E-02	6.6437E+01	1.3952E+03
500	7.9100E-03	6.1700E+01	1.7000E-02	6.1725E+01	1.2962E+03
550	7.6900E-03	5.7700E+01	1.3400E-02	5.7721E+01	1.2121E+03
600	7.5300E-03	5.4100E+01	1.2200E-02	5.4120E+01	1.1365E+03
650	7.4200E-03	5.0900E+01	1.1800E-02	5.0919E+01	1.0693E+03
700	7.3300E-03	4.8000E+01	1.1600E-02	4.8019E+01	1.0084E+03
750	7.2700E-03	4.5300E+01	1.1500E-02	4.5319E+01	9.5169E+02
800	7.2200E-03	4.2900E+01	1.1500E-02	4.2919E+01	9.0129E+02
850	7.1800E-03	4.0700E+01	1.1500E-02	4.0719E+01	8.5509E+02
900	7.1500E-03	3.8700E+01	1.1500E-02	3.8719E+01	8.1309E+02
950	7.1300E-03	3.6800E+01	1.1400E-02	3.6819E+01	7.7319E+02
1000	7.1100E-03	3.5100E+01	1.1400E-02	3.5119E+01	7.3749E+02
1500	6.9600E-03	2.3600E+01	1.1400E-02	2.3618E+01	4.9599E+02
2000	6.8400E-03	1.8100E+01	1.1400E-02	1.8118E+01	3.8048E+02
2500	6.7200E-03	1.5400E+01	1.1300E-02	1.5418E+01	3.2378E+02
3000	6.6100E-03	1.4000E+01	1.1300E-02	1.4018E+01	2.9438E+02
3500	6.5000E-03	1.3100E+01	1.1300E-02	1.3118E+01	2.7547E+02
4000	6.3800E-03	1.2400E+01	1.1200E-02	1.2418E+01	2.6077E+02
4500	6.2700E-03	1.1900E+01	1.1200E-02	1.1917E+01	2.5027E+02
5000	6.1700E-03	1.1400E+01	1.1200E-02	1.1417E+01	2.3976E+02
5500	6.0600E-03	1.1000E+01	1.1100E-02	1.1017E+01	2.3136E+02
6000	5.9600E-03	1.0700E+01	1.1100E-02	1.0717E+01	2.2506E+02
6500	5.8600E-03	1.0300E+01	1.1100E-02	1.0317E+01	2.1666E+02
7000	5.7600E-03	9.9600E+00	1.1100E-02	9.9769E+00	2.0951E+02
7500	5.6600E-03	9.6300E+00	1.1000E-02	9.6467E+00	2.0258E+02
8000	5.5600E-03	9.3200E+00	1.1000E-02	9.3366E+00	1.9607E+02
8500	5.4700E-03	9.0200E+00	1.1000E-02	9.0365E+00	1.8977E+02
9000	5.3800E-03	8.7400E+00	1.0900E-02	8.7563E+00	1.8388E+02

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9500	5.2800E-03	8.4700E+00	1.0900E-02	8.4862E+00	1.7821E+02
10000	5.1900E-03	8.2100E+00	1.0900E-02	8.2261E+00	1.7275E+02
15000	4.3800E-03	6.1300E+00	1.0600E-02	6.1450E+00	1.2904E+02
20000	3.7000E-03	4.7500E+00	1.0300E-02	4.7640E+00	1.0004E+02
25000	3.1300E-03	3.8000E+00	1.0100E-02	3.8132E+00	8.0078E+01
30000	2.6500E-03	3.1200E+00	9.8100E-03	3.1325E+00	6.5782E+01
35000	2.2400E-03	2.6200E+00	9.5600E-03	2.6318E+00	5.5268E+01
40000	1.9000E-03	2.2400E+00	9.3200E-03	2.2512E+00	4.7276E+01
45000	1.6100E-03	1.9400E+00	9.0800E-03	1.9507E+00	4.0964E+01
50000	1.3700E-03	1.7000E+00	8.8500E-03	1.7102E+00	3.5915E+01
55000	1.1600E-03	1.5000E+00	8.6300E-03	1.5098E+00	3.1706E+01
60000	9.9200E-04	1.3400E+00	8.4100E-03	1.3494E+00	2.8337E+01
65000	8.4600E-04	1.2000E+00	8.2000E-03	1.2090E+00	2.5390E+01
70000	7.2200E-04	1.0900E+00	8.0000E-03	1.0987E+00	2.3073E+01
75000	6.1800E-04	9.8800E-01	7.8000E-03	9.9642E-01	2.0925E+01
80000	5.3000E-04	9.0400E-01	7.6100E-03	9.1214E-01	1.9155E+01
85000	4.5500E-04	8.3200E-01	7.4300E-03	8.3989E-01	1.7638E+01
90000	3.9200E-04	7.7100E-01	7.2500E-03	7.7864E-01	1.6351E+01
95000	3.3900E-04	7.1900E-01	7.0700E-03	7.2641E-01	1.5255E+01
100000	2.9400E-04	6.7400E-01	6.9000E-03	6.8119E-01	1.4305E+01
150000	9.0900E-05	4.7000E-01	5.4300E-03	4.7552E-01	9.9859E+00
200000	4.8600E-05	4.2800E-01	4.3300E-03	4.3238E-01	9.0800E+00
250000	3.7100E-05	4.1700E-01	3.4800E-03	4.2052E-01	8.8309E+00
300000	3.2400E-05	4.0800E-01	2.8300E-03	4.1086E-01	8.6281E+00
350000	2.9600E-05	3.9700E-01	2.3300E-03	3.9936E-01	8.3866E+00
400000	2.7600E-05	3.8400E-01	1.9400E-03	3.8597E-01	8.1053E+00
450000	2.6000E-05	3.7000E-01	1.6300E-03	3.7166E-01	7.8048E+00
500000	2.4700E-05	3.5600E-01	1.3800E-03	3.5740E-01	7.5055E+00
550000	2.3700E-05	3.4200E-01	1.1800E-03	3.4320E-01	7.2073E+00
600000	2.2800E-05	3.2900E-01	1.0200E-03	3.3004E-01	6.9309E+00
650000	2.1900E-05	3.1700E-01	8.9000E-04	3.1791E-01	6.6761E+00
700000	2.1200E-05	3.0500E-01	7.8200E-04	3.0580E-01	6.4219E+00
750000	2.0500E-05	2.9500E-01	6.9300E-04	2.9571E-01	6.2100E+00
800000	1.9900E-05	2.8500E-01	6.1900E-04	2.8564E-01	5.9984E+00
850000	1.9300E-05	2.7600E-01	5.5700E-04	2.7658E-01	5.8081E+00
900000	1.8700E-05	2.6700E-01	5.0400E-04	2.6752E-01	5.6180E+00
950000	1.8200E-05	2.6000E-01	4.6000E-04	2.6048E-01	5.4700E+00
1000000	1.7700E-05	2.5300E-01	4.2200E-04	2.5344E-01	5.3222E+00

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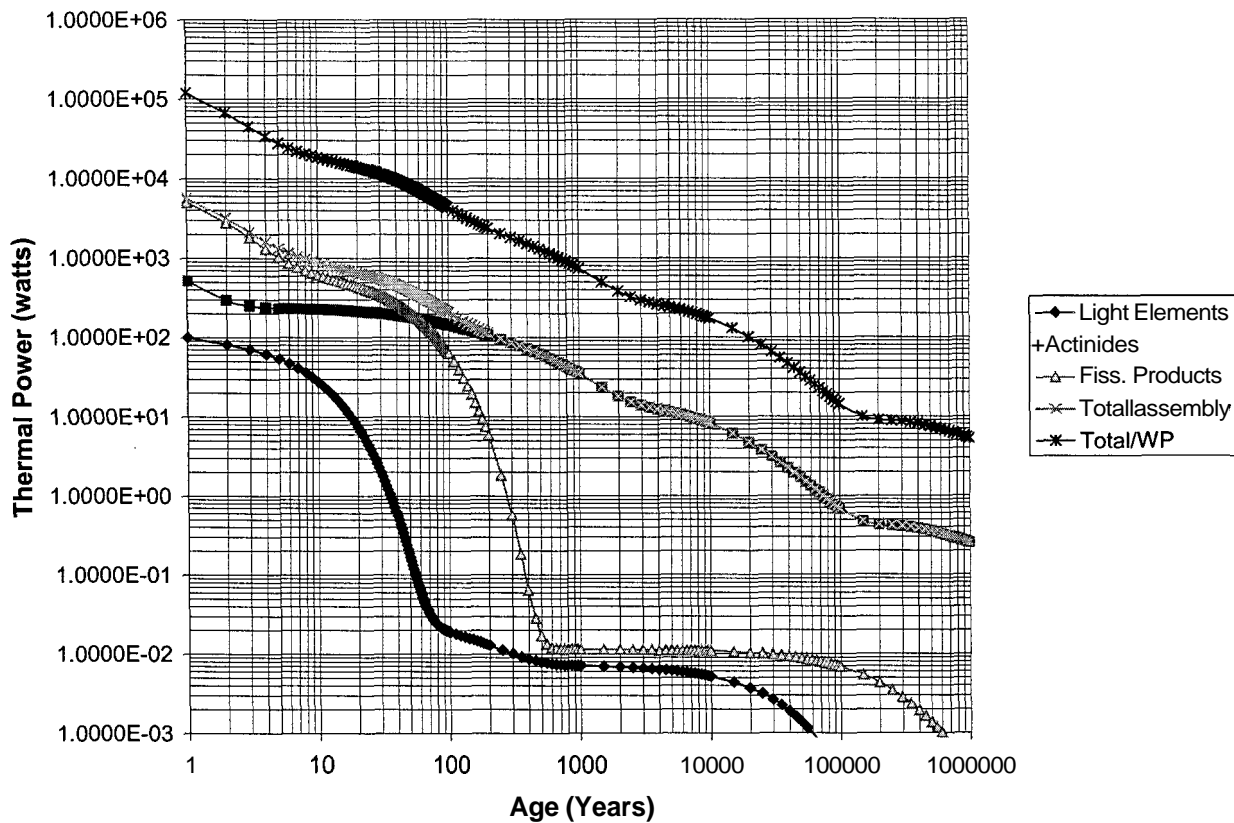


Figure 6. Thermal Power of the Average PWR SNF Assembly

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## 8. ATTACHMENTS

The list of attachments is presented in Table 24. Electronic output files are provided on a compact disk that is Attachment III of this calculation. The files' attributes of the output files are listed in Table 25. Each file is identified by its name, size (in bytes), and the date and time. The files can be read using Word-Pad application in Microsoft Windows Explorer. It should be noted that the date and time reflect the time of transfer from the HP to the personal computer. The actual date and time of each run completion can be found in its corresponding file.

Table 24. List of Attachments

Description	Attachment Number	No. of Pages
SK - 0175 REV 02 21-PWR Waste Package Configurations for Site Recommendation	I	2
SK - 0191 REV 00 21-PWR Waste Package Weld Configuration	II	1
Compact Disk of MCNP Output and Excel Files	III	N/A

Table 25. Files in the Compact Disk

File Name	Description	Size (bytes)	Date	Time
1g10o	MCNP gamma case at RH=40% & 90° C, 10 years SNF	214,683	7/25/2002	10:38a
1g15o	MCNP gamma case at RH=40% & 90° C, 15 years SNF	1206,287	7/25/2002	10:39a
1g20o	MCNP gamma case at RH=40% & 90° C, 20 years SNF	206,153	7/25/2002	10:39a
1g25o	MCNP gamma case at RH=40% & 90° C, 25 years SNF	206,165	7/25/2002	10:39a
1g35o	MCNP gamma case at RH=40% & 90° C, 35 years SNF	1206,165	7/25/2002	10:39a
1g50o	MCNP gamma case at RH=40% & 90° C, 50 years SNF	206,165	7/25/2002	10:39a
1g70o	MCNP gamma case at RH=40% & 90° C, 70 years SNF	206,287	7/25/2002	10:39a
1g100o	MCNP gamma case at RH=40% & 90° C, 100 years SNF	206,153	7/25/2002	04:12p
1g150o	MCNP gamma case at RH=40% & 90° C, 150 years SNF	220,202	7/25/2002	04:12p
1g200o	MCNP gamma case at RH=40% & 90° C, 200 years SNF	220,993	7/25/2002	04:12p
1g250o	MCNP gamma case at RH=40% & 90° C, 250 years SNF	221,396	7/25/2002	04:12p
1g350o	MCNP gamma case at RH=40% & 90° C, 350 years SNF	207,084	7/25/2002	04:13p
1g500o	MCNP gamma case at RH=40% & 90° C, 500 years SNF	218,162	7/25/2002	04:13p
1g700o	MCNP gamma case at RH=40% & 90° C, 700 years SNF	214,102	7/25/2002	04:13p
1g1.0ko	MCNP gamma case at RH=40% & 90° C, 1000 years SNF	218,426	7/25/2002	04:11p
1g1.5ko	MCNP gamma case at RH=40% & 90° C, 1500 years SNF	205,884	7/25/2002	04:12p
1g2ko	MCNP gamma case at RH=40% & 90° C, 2000 years SNF	207,884	7/25/2002	04:12p
1g2.5ko	MCNP gamma case at RH=40% & 90° C, 2500 years SNF	1208,684	7/25/2002	04:12p
1g3.5ko	MCNP gamma case at RH=40% & 90° C, 3500 years SNF	207,084	7/25/2002	04:12p
1g5ko	MCNP gamma case at RH=40% & 90° C, 5000 years SNF	207,472	7/25/2002	04:13p
1g7ko	MCNP gamma case at RH=40% & 90° C, 7000 years SNF	207,472	7/25/2002	04:13p
1g10ko	MCNP gamma case at RH=40% & 90° C, 10000 years SNF	208,284	7/25/2002	04:12p
1g15ko	MCNP gamma case at RH=40% & 90° C, 15000 years SNF	219,635	7/25/2002	04:12p
1g20ko	MCNP gamma case at RH=40% & 90° C, 20000 years SNF	214,790	7/25/2002	04:12p
1g25ko	MCNP gamma case at RH=40% & 90° C, 25000 years SNF	208,205	7/25/2002	04:12p
1g35ko	MCNP gamma case at RH=40% & 90° C, 35000 years SNF	1209,500	7/25/2002	10:37a
1g50ko	MCNP gamma case at RH=40% & 90° C, 50000 years SNF	1222,310	7/25/2002	04:13p

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1g70ko	MCNP gamma case at RH=40% & 90° C, 70000 years SNF	1208,309	7/25/2002	04:13p
1g100ko	MCNP gamma case at RH=40% & 90° C, 100000 years SNF	205,918	7/25/2002	04:12p
1g150ko	MCNP gamma case at RH=40% & 90° C, 150000 years SNF	208,855	7/25/2002	04:12p
1g200ko	MCNP gamma case at RH=40% & 90° C, 200000 years SNF	208,855	7/25/2002	04:12p
1g250ko	MCNP gamma case at RH=40% & 90° C, 250000 years SNF	208,966	7/25/2002	04:12p
1g350ko	MCNP gamma case at RH=40% & 90° C, 350000 years SNF	1208,309	7/25/2002	04:13p
1g500ko	MCNP gamma case at RH=40% & 90° C, 500000 years SNF	205,650	7/25/2002	04:13p
1g700ko	MCNP gamma case at RH=40% & 90° C, 700000 years SNF	208,954	7/25/2002	04:13p
1g1mo	MCNP gamma case at RH=40% & 90° C, 1000000 years SNF	208,420	7/25/2002	04:12p
2g10o	MCNP gamma case at RH=40% & 30° C, 10 years SNF	208,879	7/25/2002	10:38a
2g15o	MCNP gamma case at RH=40% & 30° C, 15 years SNF	220,311	7/25/2002	10:38a
2g20o	MCNP gamma case at RH=40% & 30° C, 20 years SNF	220,980	7/25/2002	10:38a
2g25o	MCNP gamma case at RH=40% & 30° C, 25 years SNF	214,517	7/25/2002	10:38a
2g35o	MCNP gamma case at RH=40% & 30° C, 35 years SNF	221,247	7/25/2002	10:38a
2g50o	MCNP gamma case at RH=40% & 30° C, 50 years SNF	208,855	7/25/2002	10:38a
2g70o	MCNP gamma case at RH=40% & 30° C, 70 years SNF	208,855	7/25/2002	10:38a
2g100o	MCNP gamma case at RH=40% & 30° C, 100 years SNF	208,855	7/25/2002	04:09p
2g150o	MCNP gamma case at RH=40% & 30° C, 150 years SNF	220,202	7/25/2002	04:09p
2g200o	MCNP gamma case at RH=40% & 30° C, 200 years SNF	220,993	7/25/2002	04:09p
2g250o	MCNP gamma case at RH=40% & 30° C, 250 years SNF	1221,396	7/25/2002	04:10p
2g350o	MCNP gamma case at RH=40% & 30° C, 350 years SNF	207,084	7/25/2002	04:10p
2g500o	MCNP gamma case at RH=40% & 30° C, 500 years SNF	218,162	7/25/2002	04:10p
2g700o	MCNP gamma case at RH=40% & 30° C, 700 years SNF	220,164	7/25/2002	04:11p
2g1.0ko	MCNP gamma case at RH=40% & 30° C, 1000 years SNF	218,426	7/25/2002	04:08p
2g1.5ko	MCNP gamma case at RH=40% & 30° C, 1500 years SNF	205,884	7/25/2002	04:08p
2g2ko	MCNP gamma case at RH=40% & 30° C, 2000 years SNF	207,884	7/25/2002	04:10p
2g2.5ko	MCNP gamma case at RH=40% & 30° C, 2500 years SNF	208,684	7/25/2002	04:09p
2g3.5ko	MCNP gamma case at RH=40% & 30° C, 3500 years SNF	207,084	7/25/2002	04:10p
2g5ko	MCNP gamma case at RH=40% & 30° C, 5000 years SNF	207,472	7/25/2002	04:10p
2g7ko	MCNP gamma case at RH=40% & 30° C, 7000 years SNF	207,484	7/25/2002	04:11p
2g10ko	MCNP gamma case at RH=40% & 30° C, 10000 years SNF	208,284	7/25/2002	04:09p
2g15ko	MCNP gamma case at RH=40% & 30° C, 15000 years SNF	219,635	7/25/2002	04:09p
2g20ko	MCNP gamma case at RH=40% & 30° C, 20000 years SNF	214,790	7/25/2002	04:10p
2g25ko	MCNP gamma case at RH=40% & 30° C, 25000 years SNF	208,205	7/25/2002	04:10p
2g35ko	MCNP gamma case at RH=40% & 30° C, 35000 years SNF	209,500	7/25/2002	10:37a
2g50ko	MCNP gamma case at RH=40% & 30° C, 50000 years SNF	222,310	7/25/2002	04:10p
2g70ko	MCNP gamma case at RH=40% & 30° C, 70000 years SNF	208,309	7/25/2002	04:11p
2g100ko	MCNP gamma case at RH=40% & 30° C, 100000 years SNF	205,930	7/25/2002	04:09p
2g150ko	MCNP gamma case at RH=40% & 30° C, 150000 years SNF	208,855	7/25/2002	04:09p
2g200ko	MCNP gamma case at RH=40% & 30° C, 200000 years SNF	208,855	7/25/2002	04:09p
2g250ko	MCNP gamma case at RH=40% & 30° C, 250000 years SNF	208,966	7/25/2002	04:10p
2g350ko	MCNP gamma case at RH=40% & 30° C, 350000 years SNF	208,309	7/25/2002	04:10p
2g500ko	MCNP gamma case at RH=40% & 30° C, 500000 years SNF	205,650	7/25/2002	04:10p
2g700ko	MCNP gamma case at RH=40% & 30° C, 700000 years SNF	208,954	7/25/2002	04:11p
2g1mo	MCNP gamma case at RH=40% & 30° C, 1000000 years SNF	208,420	7/25/2002	04:09p
3g10o	MCNP gamma case at RH=90% & 30° C, 10 years SNF	208,879	7/25/2002	10:39a
3g15o	MCNP gamma case at RH=90% & 30° C, 15 years SNF	214,517	7/25/2002	10:39a
3g20o	MCNP gamma case at RH=90% & 30° C, 20 years SNF	208,588	7/25/2002	10:40a

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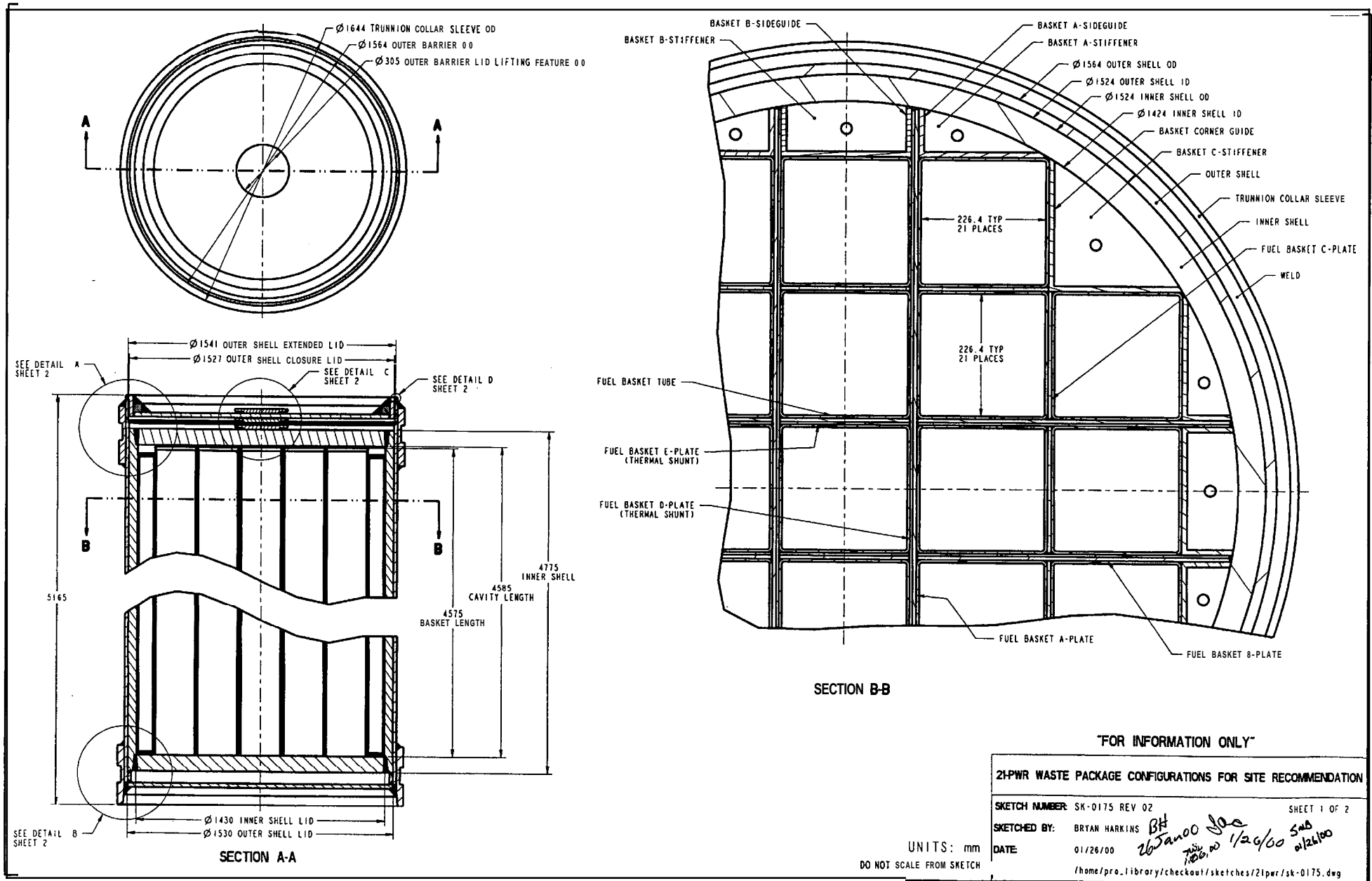
3g25o	MCNP gamma case at RH=90% & 30° C, 25 years SNF	214,517	7/25/2002	10:40a
3g35o	MCNP gamma case at RH=90% & 30" C, 35 years SNF	221,247	7/25/2002	10:40a
3g50o	MCNP gamma case at RH=90% & 30" C, 50 years SNF	208,855	7/25/2002	10:40a
3g70o	MCNP gamma case at RH=90% & 30° C, 70 years SNF	1208,855	7/25/2002	10:40a
3g100o	MCNP gamma case at RH=90% & 30° C, 100 years SNF	208,855	7/25/2002	04:19p
3g150o	MCNP gamma case at RH=90% & 30° C, 150 years SNF	220,202	7/25/2002	04:21p
3g200o	MCNP gamma case at RH=90% & 30" C, 200 years SNF	220,993	7/25/2002	04:21p
3g250o	MCNP gamma case at RH=90% & 30° C, 250 years SNF	1221,396	7/25/2002	04:21p
3g350o	MCNP gamma case at RH=90% & 30° C, 350 years SNF	207,084	7/25/2002	04:22p
3g500o	MCNP gamma case at RH=90% & 30° C, 500 years SNF	218,162	7/25/2002	04:22p
3g700o	MCNP gamma case at RH=90% & 30" C, 700 years SNF	214,102	7/25/2002	04:22p
3g1.0ko	MCNP gamma case at RH=90% & 30° C, 1000 years SNF	218,426	7/25/2002	04:14p
3g1.5ko	MCNP gamma case at RH=90% & 30° C, 1500 years SNF	205,884	7/25/2002	04:14p
3g2ko	MCNP gamma case at RH=90% & 30° C, 2000 years SNF	207,884	7/25/2002	04:22p
3g2.5ko	MCNP gamma case at RH=90% & 30° C, 2500 years SNF	208,684	7/25/2002	04:21p
3g3.5ko	MCNP gamma case at RH=90% & 30" C, 3500 years SNF	207,084	7/25/2002	04:22p
3g5ko	MCNP gamma case at RH=90% & 30° C, 5000 years SNF	207,472	7/25/2002	04:22p
3g7ko	MCNP gamma case at RH=90% & 30" C, 7000 years SNF	207,472	7/25/2002	04:23p
3g10ko	MCNP gamma case at RH=90% & 30" C, 10000 years SNF	208,284	7/25/2002	04:21p
3g15ko	MCNP gamma case at RH=90% & 30° C, 15000 years SNF	219,635	7/25/2002	04:21p
3g20ko	MCNP gamma case at RH=90% & 30" C, 20000 years SNF	214,790	7/25/2002	04:21p
3g25ko	MCNP gamma case at RH=90% & 30" C, 25000 years SNF	208,205	7/25/2002	04:22p
3g35ko	MCNP gamma case at RH=90% & 30" C, 35000 years SNF	209,500	7/25/2002	10:37a
3g50ko	MCNP gamma case at RH=90% & 30" C, 50000 years SNF	222,310	7/25/2002	04:22p
3g70ko	MCNP gamma case at RH=90% & 30° C, 70000 years SNF	208,309	7/25/2002	04:23p
3g100ko	MCNP gamma case at RH=90% & 30° C, 100000 years SNF	205,918	7/25/2002	04:19p
3g150ko	MCNP gamma case at RH=90% & 30" C, 150000 years SNF	208,855	7/25/2002	04:21p
3g200ko	MCNP gamma case at RH=90% & 30° C, 200000 years SNF	208,855	7/25/2002	04:21p
3g250ko	MCNP gamma case at RH=90% & 30" C, 250000 years SNF	208,966	7/25/2002	04:21p
3g350ko	MCNP gamma case at RH=90% & 30° C, 350000 years SNF	1208,309	7/25/2002	04:22p
3g500ko	MCNP gamma case at RH=90% & 30° C, 500000 years SNF	1205,650	7/25/2002	04:22p
3g700ko	MCNP gamma case at RH=90% & 30" C, 700000 years SNF	1208,954	7/25/2002	04:22p
3g1mo	MCNP gamma case at RH=90% & 30° C, 1000000 years SNF	208,420	7/25/2002	04:21p
4g10o	MCNP gamma case at RH=90% & 90° C, 10 years SNF	208,879	9/12/2002	07:45a
4g15o	MCNP gamma case at RH=90% & 90" C, 15 years SNF	208,321	9/12/2002	07:46a
4g20o	MCNP gamma case at RH=90% & 90" C, 20 years SNF	208,588	9/12/2002	07:46a
4g25o	MCNP gamma case at RH=90% & 90" C, 25 years SNF	214,517	9/12/2002	07:46a
4g35o	MCNP gamma case at RH=90% & 90" C, 35 years SNF	221,247	9/12/2002	07:47a
4g50o	MCNP gamma case at RH=90% & 90" C, 50 years SNF	208,855	9/12/2002	07:47a
4g70o	MCNP gamma case at RH=90% & 90" C, 70 years SNF	208,855	9/12/2002	07:47a
4g100o	MCNP gamma case at RH=90% & 90° C, 100 years SNF	208,855	9/12/2002	07:45a
4g150o	MCNP gamma case at RH=90% & 90° C, 150 years SNF	220,202	9/12/2002	07:46a
4g200o	MCNP gamma case at RH=90% & 90" C, 200 years SNF	220,859	9/12/2002	07:46a
4g250o	MCNP gamma case at RH=90% & 90" C, 250 years SNF	221,262	9/12/2002	07:46a
4g350o	MCNP gamma case at RH=90% & 90" C, 350 years SNF	207,096	9/12/2002	07:47a
4g500o	MCNP gamma case at RH=90% & 90" C, 500 years SNF	218,162	9/12/2002	07:47a
4g700o	MCNP gamma case at RH=90% & 90° C, 700 years SNF	220,164	9/12/2002	07:47a
4g1.0ko	MCNP gamma case at RH=90% & 90" C, 1000 years SNF	218,426	9/12/2002	07:45a

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4g1.5ko	MCNP gamma case at RH=90% & 90° C, 1500 years SNF	1205,884	9/12/2002	07:45a
4g2ko	MCNP gamma case at RH=90% & 90° C, 2000 years SNF	1207,884	9/12/2002	07:46a
4g2.5ko	MCNP gamma case at RH=90% & 90° C, 2500 years SNF	208,672	9/12/2002	07:46a
4g3.5ko	MCNP gamma case at RH=90% & 90° C, 3500 years SNF	207,084	9/12/2002	07:47a
4g5ko	MCNP gamma case at RH=90% & 90° C, 5000 years SNF	207,472	9/12/2002	07:47a
4g7ko	MCNP gamma case at RH=90% & 90° C, 7000 years SNF	207,484	9/12/2002	07:48a
4g10ko	MCNP gamma case at RH=90% & 90° C, 10000 years SNF	208,284	9/12/2002	07:45a
4g15ko	MCNP gamma case at RH=90% & 90° C, 15000 years SNF	219,635	9/12/2002	07:46a
4g20ko	MCNP gamma case at RH=90% & 90° C, 20000 years SNF	214,924	9/12/2002	07:46a
4g25ko	MCNP gamma case at RH=90% & 90° C, 25000 years SNF	208,205	9/12/2002	07:46a
4g35ko	MCNP gamma case at RH=90% & 90° C, 35000 years SNF	209,500	9/12/2002	07:47a
4g50ko	MCNP gamma case at RH=90% & 90° C, 50000 years SNF	222,444	9/12/2002	07:47a
4g70ko	MCNP gamma case at RH=90% & 90° C, 70000 years SNF	208,309	9/12/2002	07:47a
4g100ko	MCNP gamma case at RH=90% & 90° C, 100000 years SNF	205,930	9/12/2002	07:45a
4g150ko	MCNP gamma case at RH=90% & 90° C, 150000 years SNF	208,855	9/12/2002	07:45a
4g200ko	MCNP gamma case at RH=90% & 90° C, 200000 years SNF	208,855	9/12/2002	07:46a
4g250ko	MCNP gamma case at RH=90% & 90° C, 250000 years SNF	1208,966	9/12/2002	07:46a
4g350ko	MCNP gamma case at RH=90% & 90° C, 350000 years SNF	208,309	9/12/2002	07:47a
4g500ko	MCNP gamma case at RH=90% & 90° C, 500000 years SNF	205,650	9/12/2002	07:47a
4g700ko	MCNP gamma case at RH=90% & 90° C, 700000 years SNF	208,954	9/12/2002	07:47a
4g1mo	MCNP gamma case at RH=90% & 90° C, 1000000 years SNF	208,420	9/12/2002	07:46a
n-23.io	MCNP neutron case at RH=40% & 30° C, per neutron	215,152	7/25/2002	04:33p
n-24.io	MCNP neutron case at RH=40% & 90° C, per neutron	215,178	7/25/2002	04:33p
n-25.io	MCNP neutron case at RH=90% & 30° C, per neutron	215,178	7/25/2002	04:34p
n-26.io	MCNP neutron case at RH=90% & 90° C, per neutron	215,178	7/25/2002	04:34p
Gamma-ED2.xls	Gamma energy depositions and dose rates for 4 moist airs	57,856	9/8/2002	08:29p
Neutron-ED.xls	Neutron energy depositions and dose rates for 4 moist airs	74,752	9/9/2002	02:03p
G-Source-4%48gwd.xls	Gamma source spectra of a PWR SNF at 4% initial enrichment and 48 GWd/MTU to one million years	273,920	9/6/2002	04:49p
N-Source-4%48gwd.xls	Neutron source spectra of a PWR SNF at 4% initial enrichment and 48 GWd/MTU to one million years	37,888	8/01/2002	01:37p
DecayPower-4%48gwd.xls	Decay heat rate to one million years	387,584	8/03/2002	10:07a
JoonLee.xls	Neutron and gamma intensities to one million years	66,048	7/25/2002	10:20a
MoistAirProperties.xls	Moist air properties and atom number densities	100,864	9/9/2002	03:13p

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345

100

125

44

100

25

45°

R5 TYP

45° TYP

45°

70 SUPPORT RING

25 OUTER SHELL FLAT BOTTOM LID

20

20

5°

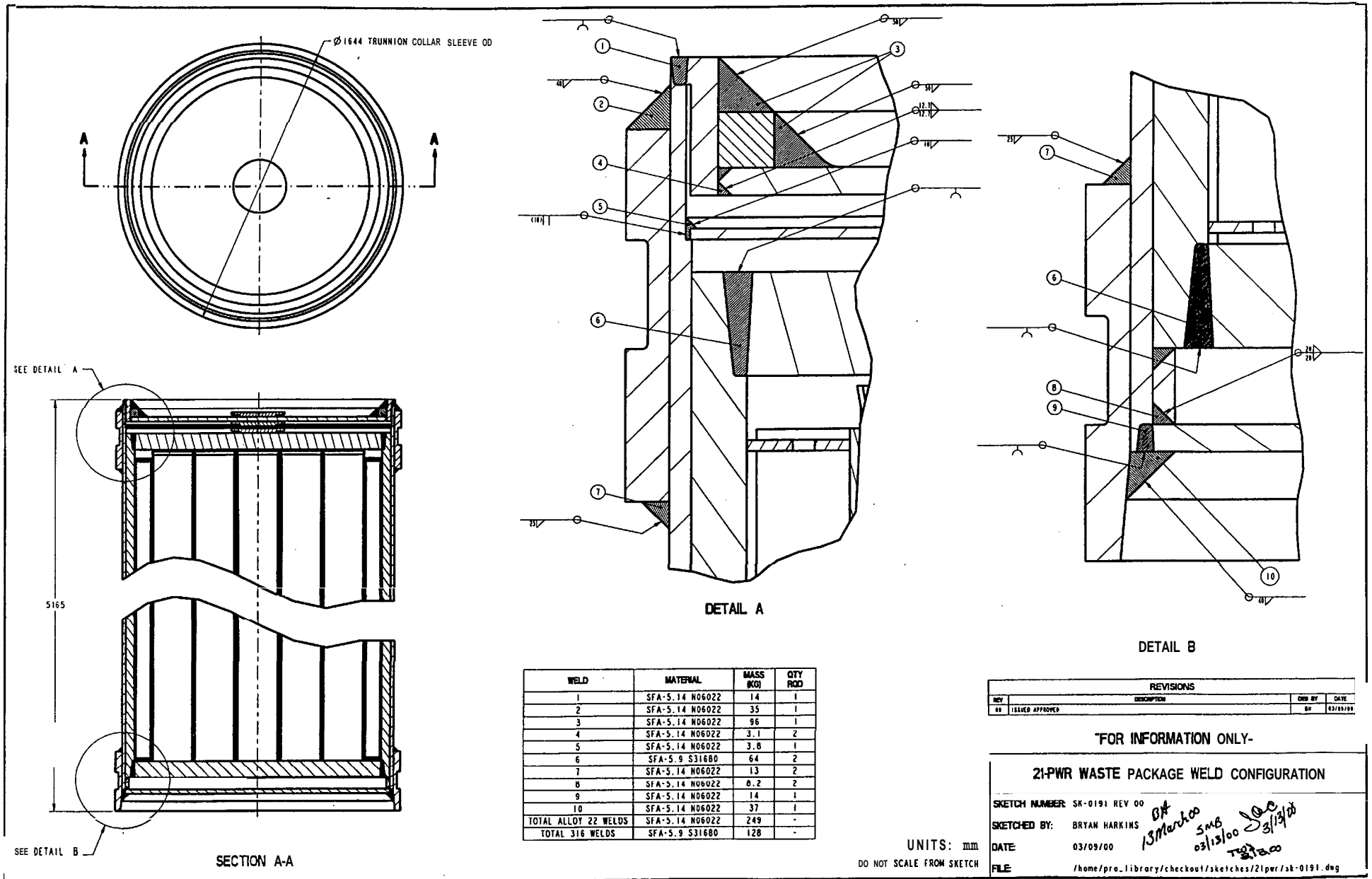
DETAIL B

I\* REFER TO SK-0191 REV 00 '21-PWR WASTE PACKAGE WELD CONFIGURATION"

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